NRC FORM 195 (2-76)

U.S. NUCLEAR REGULATORY COMMISSION

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DESCRIPTION

Ltr furnishing info concerning modification to question 5.37 which will be included with material submitted in Amdt #16 to the PSAR....

ENCLOSURE

DO NOT REMOVE

ACKNOWLEDGED

PLANT NAME: Susquehanna 1 & 2

SAFETY — FOR ACTION/INFORMATION — ENVIRO 3-31-76 enf

	SAFETY		ENVIRO
✓	ASSIGNED AD :	De Young	ASSIGNED AD :
✓	BRANCH CHIEF :	Butler	BRANCH CHIEF :
✓	PROJECT MANAGER:	Minor	PROJECT MANAGER :
✓	LIC. ASST. :	Rushbrook	LIC. ASST. :

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✓	I & E (2)		SCHROEDER	✓	BENAROYA		BALLARD
✓	OELD			✓	LAINAS		SPANGLER
	GOSSICK & STAFF		ENGINEERING	✓	IPPOLITO		
	MIPC	✓	MACCARY				SITE TECH
	CASE	✓	KNIGHT		OPERATING REACTORS	✓	GAMMILL (2)
	HANAUER	✓	SIHWEIL	✓	STELLO		STEPP
	HARLESS	✓	PAWLICKI				HULMAN
					OPERATING TECH		
	PROJECT MANAGEMENT		REACTOR SAFETY		EISENHUT		SITE ANALYSIS
	BOYD	✓	ROSS		SHAO	✓	VOLLMER
	P. COLLINS	✓	NOVAK		BAER	✓	BUNCH
	HOUSTON	✓	ROSZTOCZY		SCHWENCER	✓	J. COLLINS
	PETERSON	✓	CHECK		GRIMES	✓	KREGER
	MELTZ						
	HELTEMES		AT & I		SITE SAFETY & ENVIRO		
	SKOVHOLT		SALTZMAN		ANALYSIS		
			RUTBERG		DENTON & MULLER		

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✓	TIC		REG. V-IE		ULRIKSON(ORNL)
✓	NSIC		LA PDR		
	ASLB		CONSULTANTS		
✓	ACRS 16 ~~HOLDING~~/SENT TO LA Rushbrook				

CONTROL NUMBER: 3112

PP&L

TWO NORTH NINTH STREET, ALLENTOWN, PA. 18101 PHONE: (215) 821-5151

March 23, 1976

Director of Nuclear Reactor Regulation
Light Water Reactors Branch No. 1-2
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Attention: Dr. Walter R. Butler, Chief

Docket Nos. 50-387
50-388

SUSQUEHANNA STEAM ELECTRIC STATION
MODIFICATION TO QUESTION 5.37
ER 100450 FILE 840-2
PLA-110

Dear Dr. Butler:

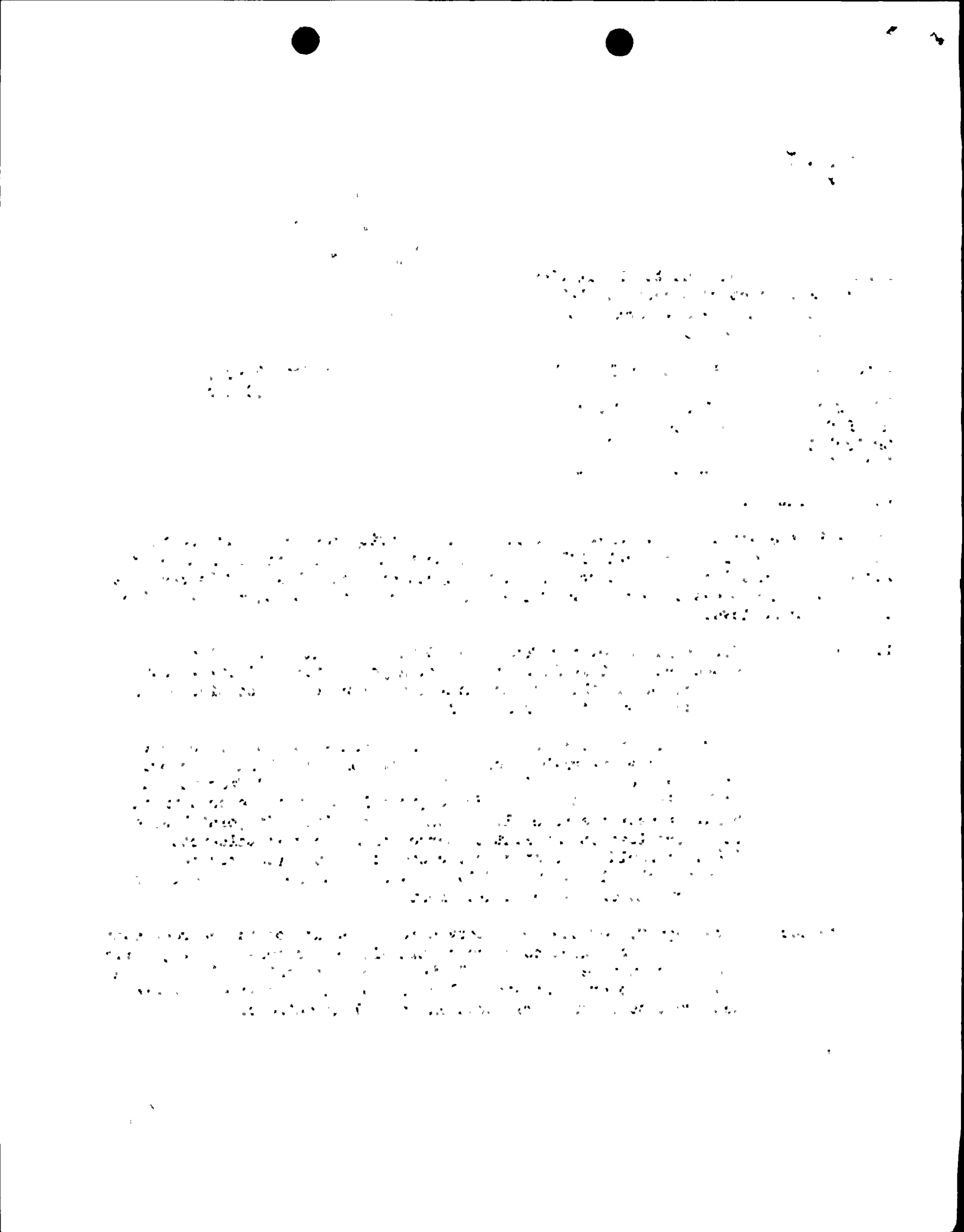
We are in the process of writing our Structural Integrity Test Procedure to be submitted to you in late April, 1976. In Amendment #16, we submitted an amended response to Question 5.37 on compliance with Regulatory Guide 1.18. This response listed our exceptions to Regulatory Guide 1.18. We have two additional exceptions. They are as follows:

1. Reference: Paragraph C.2 of the Regulatory Guide. We intend to select the number and distribution of measuring points for monitoring radial deflections so that the as-built conditions can be considered in the assessment of general shell response.

   In general the locations of measuring points for radial deflections are in agreement with Figure B, except point 1. Point 1 is provided at a distance of two times the wall thickness (12') from the base slab. This variation is made to properly predict the containment behavior near the base slab to wall connection. If point 1 were to be provided at a height of three times the wall thickness (18') it would be located very close to point 2 (suppression chamber wall midheight is 26') and would not yield any additional behavior pattern of the containment.

2. Reference: Paragraph C.5 of the Regulatory Guide. Because of the current state of the art, triaxial concrete strain distortion measurements, while taken, may not be used to evaluate the containment strain distribution. The concrete strain will be evaluated using linear strain measurements in the meridional and radial directions.

3112



As we stated before, we intend to submit our Structural Integrity Test Procedure in late April and would like your comments on these exceptions prior to that date.

Very truly yours,

N. W. Curtis
Vice President-Engineering & Construction

NWC:AAW