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TO: DIR NRC

FROM: PP&L  
ALLENTOWN, PA  
N W CURTIS.....

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**9-9-76**

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**9-13-76**

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DESCRIPTION  
LTR REF OUR 8-9-76 LTR....FURNISHING RESPONSE  
TO A QUESTION ON ANNULUS PRESSURIZATION AS  
THAT INFO WILL BE A PART OF THE FSAR REVIEW  
AND FURTHER RESPONDING TO QUET QUESTION  
CONCERNING CRACKS IN THE FEEDWATER NOZZLE  
BLEND RADII.....

ENCLOSURE

**DO NOT REMOVE**

**ACKNOWLEDGED**

PLANT NAME: **SUSQUEHANA 1:2**

SAFETY FOR ACTION/INFORMATION ENVIRO **9-15-76 RKB**

ASSIGNED AD:		ASSIGNED AD:
BRANCH CHIEF:	<b>(6) PARR</b>	BRANCH CHIEF:
PROJECT MANAGER:	<b>MINER</b>	PROJECT MANAGER:
LIC. ASST.:	<b>(17) RUSHBROOK</b>	LIC. ASST.:

INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> REG FILE		SYSTEMS SAFETY		PLANT SYSTEMS		SITE SAFETY &
<input checked="" type="checkbox"/> NRC PDR		HEINEMAN		TEDESCO		ENVIRO ANALYSIS
<input checked="" type="checkbox"/> I & E		SCHROEDER		BENAROYA		DENTON & MULLER
<input checked="" type="checkbox"/> OELD				LAINAS		
GOSSICK & STAFF		ENGINEERING		IPPOLITO		ENVIRO TECH.
MIPC		MACCARRY		KIRKWOOD		ERNST
CASE		KNIGHT				BALLARD
HANAUER		SIHWEIL		OPERATING REACTORS		SPANGLER
HARLESS		PAWLICKI		STELLO		
PROJECT MANAGEMENT		REACTOR SAFETY		OPERATING TECH.		SITE TECH.
BOYD		ROSS		EISENHUT		GAMMILL (2)
P. COLLINS		NOVAK		SHAO		STEPP
HOUSTON		ROSZTOCZY		BAER		HULMAN
PETERSON		CHECK		BUTLER		SITE ANALYSIS
MELTZ				GRIMES		VOLLMER
HELTEMES		AT & I				BUNCH
SKOVHOLT		SALTZMAN				J. COLLINS
		RUTBERG				KREGER

EXTERNAL DISTRIBUTION			CONTROL NUMBER
<input checked="" type="checkbox"/> LPDR: WILKES BARRE PA	NAT LAB:	BROOKHAVEN NAT LAB	<b>9302</b>
<input checked="" type="checkbox"/> TIC:	REG. VIE	ULRIKSON(ORNL)	
<input checked="" type="checkbox"/> NSIC:	LA PDR		
<input checked="" type="checkbox"/> ASLB:	CONSULTANTS		
<input checked="" type="checkbox"/> ACRS 16CYS HOLDING (SENT)	To L.A.		





So your children can tell their children.



SEP 09 1976

Director of Nuclear Reactor Regulation  
Light Water Reactors Branch No. 3  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

**Regulatory Docket File**

Attention: Olan D. Parr, Chief

SUSQUEHANNA STEAM ELECTRIC STATION  
RESPONSE TO LETTER DATED 8/9/76  
ER 100450                      FILE 840-2  
PIA-131

Dear Mr. Parr:

Pennsylvania Power & Light Company has the following response to your letter of August 9, 1976:

1. A response to the question on annulus pressurization will be provided in conjunction with the FSAR effort. We intend to review our methodology with you as part of the FSAR review.
2. In response to the question concerning the cracks in the feedwater, nozzle blend radii, we have the following response:

General Electric has undertaken an extensive test program to investigate the thermal cycling and associated nozzle blend radii cracks which have occurred on reactor vessel feedwater nozzles. The program includes facilities which can reproduce the thermal cycling phenomena which will allow evaluation of proposed modifications to ensure thermal cycling is reduced. Testing is scheduled for completion in mid 1977. Based on current test data and evaluations, a welded thermal sleeve appears to offer the most positive means for correcting the problem.

Additionally, in-service inspection of the nozzle inside radius section of the RPV feedwater inlet nozzles will be accomplished by ultrasonic examination from the exterior surfaces in accordance with ASME code requirements.

9302



Director of Nuclear Reactor Regulation  
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If the investigation of this matter indicates that additional protective measures are warranted, design modifications and/or supplemental in-service inspections will be implemented as appropriate.

Very truly yours,

A handwritten signature in cursive script, appearing to read "N. W. Curtis".

N. W. Curtis  
Vice President-Engineering & Construction

CTC:CAW

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