

APPENDIX C

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the Matter of )

TENNESSEE VALLEY AUTHORITY )

(Browns Ferry Nuclear Power Facility) )

Docket Nos. 50-259/260/296

ORDER MODIFYING LICENSE EFFECTIVE IMMEDIATELY

I

The Tennessee Valley Authority (the "licensee") is the holder of Operating Licenses DPR-33, DRP-52 and DPR-68 (the "licenses") which authorize the operation of the Browns Ferry Nuclear Plant, Units 1, 2 and 3, at steady state reactor core power levels not in excess of 3293 megawatts thermal (rated power) each. The licenses were issued on June 26, 1973, June 28, 1974 and July 2, 1976. The facility consists of three light water moderated, boiling water reactors (BWRs) located at the licensee's site in Limestone County, Alabama, on the Tennessee River.

II

At 6:45 a.m. on December 6, 1979, following a scheduled refueling outage the Unit 3 reactor was taken critical. Pressurization of the drywell to 1.35 psig was completed at 12:35 p.m. on December 8, 1979. Containment pressure decreased to atmospheric in about two hours. On December 9, 1979, at 2:50 a.m. after repair to a defective solenoid valve in the nitrogen makeup system and an attempt to cycle the torus-to-drywell vacuum breakers, the drywell was repressurized. When nitrogen addition was stopped, the drywell pressure again rapidly decreased. At about

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4:30 a.m., the southeast drywell equipment hatch was found to be leaking excessively. During the period from return to critical at 6:45 a.m. on December 6, 1979, until containment integrity was restored at 8:30 a.m. on December 9, 1979, following discovery and retightening of a leaking equipment hatch, the facility was operated in noncompliance with the Limiting Condition for Operation specified in Technical Specification 3.7.A.2. This hatch had been removed and reinstalled during the refueling outage without benefit of a written procedure, as required by Technical Specification 6.3.A.1. In addition, this matter was not reported to the NRC within the time specified in Technical Specification 6.7.2.a.

The specific items of noncompliance associated with the incident which are described in the Notice of Violation issued this date demonstrate that the licensee has not adopted appropriate controls to assure that maintenance activities do not defeat required safety features. There is reasonable assurance that containment closures for the Browns Ferry Units are in their proper condition at this time. However, it is also necessary to assure that future maintenance activities will be performed in a manner that will not defeat required safety features. Therefore, I have determined that the public health, safety, and interest requires, an immediately effective, modification of License Nos. DPR-33, DPR-52 and DPR-68 as stated in Part III of this Order.

## III

Accordingly, pursuant to the Atomic Energy Act of 1954 as amended, and the Commission's regulations in 10 CFR Parts 2 and 50, Licenses DPR-33, DRP-52 and DPR-68 are modified, effective immediately, as follows:

- (1) Administrative Procedures shall be adopted and implemented to require independent verification of the restoration of containment penetration closures to their normal condition after the performance of any maintenance work or any other activity which could affect the integrity of the penetration.
- (2) Existing procedures and controls shall be reviewed and revised as necessary to assure that limiting conditions for operation are not defeated by maintenance or other activities.
- (3) Procedures shall be prepared and implemented for removal and replacement of all containment penetration closures.
- (4) Procedures for reporting shall be reviewed and revised as necessary to assure compliance with all reporting requirements.
- (5) Actions required by paragraphs (1) through (4) above shall be completed and a report submitted by January 15, 1980, to the Director of NRC's Region II Office.



- (6) The licensee shall meet with Office of Inspection and Enforcement management, on or before January 21, 1980, in a meeting open to the public in the vicinity of the Browns Ferry site to describe how the above requirements will be implemented. The Director, Region II, will inform the licensee at least one week in advance of the specific time and location of the meeting.

## IV

The licensee, or any other person who has an interest affected by this Order, may, within twenty-five days of the date of this Order, request a hearing. A request for a hearing shall be addressed to the Director, Office of Inspection and Enforcement U.S.N.R.C., Washington, D.C. 20555. If a hearing is requested by the licensee or an interested person, the Commission will issue an Order designating the time and place of hearing. Such a request for hearing SHALL NOT STAY THE IMMEDIATE EFFECTIVENESS OF THIS ORDER.

Continuation of operation on terms consistent with this Order is not stayed by the pendency of any proceeding on this Order. In the event that a need for further enforcement action becomes apparent, either in the course of any proceeding on this Order or at any other time, the Director will take appropriate action.

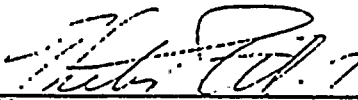


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In the event the licensee or any other interested person requests a hearing as provided above and a hearing is held, the issues to be considered at such a hearing shall be:

- (1) whether the facts set forth in Part II of this Order are correct; and,
- (2) whether this Order should be sustained.

FOR THE NUCLEAR REGULATORY COMMISSION



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Victor Stello, Jr.  
Director  
Office of Inspection  
and Enforcement

Dated at Bethesda, Maryland  
this 4th day of January 1980



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