Dec MRO-PORM 336 NUCLEAR REGULATORY COMMISSION {7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)](י)[10101010 ⁷3]3]7 - 10 0000 1 5 BR 10 10 LICENSE NUMBER LICENSEE CODE CON'T REPORT 6 0 0 1 2 0 5 7 9 8 0 1 0 4 8 0 9 68 69 EVENT DATE 74 75 REPORT DATE 80 L 6 0 5 0 0 0 0 1 DOCKET NUMBER EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) During normal operation while performing SI 4.2.A-7, Main Steam Line High Flow, switch 0 2 PdIS-1-50D exceeded the technical specification limit (T.S. Table 3.2.A). Redundant 03 switches were operable. There was no hazard to the public health or safety. Previous 04 259/7410. events: 0 5 0 6 0 7 0 8 CAUSE CODE CAUSE SYSTEM CODE COMP. VALVE SUBCODE COMPONENT CODE SUBCODE E (12) E (13) | I N S | T | R | U | (14) | S | (15) SH (1) 0 9 | Z | (16) 13 10 18 12 20 REVISION SEQUENTIAL OCCURRENCE REPORT LER/RO EVENT YEAR REPORT NO. CODE TYPE NO. 17 REPORT NUMBER 7 9 0 2 5 03 0 Τ. 22 28 32 PRIME COMP. COMPONENT ACTION FUTURE TAKEN ACTION EFFECT ON PLANT SHUTDOWN METHOD ATTACHMENT SUBMITTED NPRD-4 HOURS (22) FORM SUB. SUPPLIER MANUFACTURER 1@ ¥_23 L 25 Z (1) 0 0 0 0 N]@ B 0 6 0 26 E 18 Z 19 Z 35 36 17 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) 10 The Model 278 switch had drifted. The switch was inspected, recalibrated and function-ally tested for repeatability. This problem appears to be random in nature: there fore, no recurrence control action is planned. 1 2 13 14 8 9 80 FACILITY OTHER STATUS METHOD OF DISCOVERY % POWER DISCOVERY DESCRIPTION (32) 15 E 28 0 9 0 29 B (31) Surveillance testing NA ACTIVITY CONTENT 80 8 13 AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE RELEASED_OF RELEASE $\begin{array}{c|c}1 & \hline \\7 & 8 \\7 & 8 \end{array} \begin{array}{c}Z & \hline \\3 \\7 & 10 \end{array} \begin{array}{c}Z & \hline \\10 \\7 \\10 \end{array} \begin{array}{c}34\\10 \\10 \end{array}$ NA NA 10 11 PERSONNEL EXPOSURES 45 44 80 DESCRIPTION (39) NUMBER TYPE NA 7 PERSONNEL INJURIES 13 80 1 8 9 11 12 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION 80 1 9 NA PUBLICITY 80 DESCRIPTION (45) NRC USE ONLY N 44 0 NA 2 10 68 69 80 100 NAME OF PREPARER. PHONE:. 80.01 09a 2.1.1

Tennessee Valley Authority Browns Ferry Nuclear Plant Form BF-17 PF 15.2 1/10/79

LER SUPPLEMENTAL INFORMATION

While performing SI 4.2.A-7, one out of four main steam line high flow switches was found to exceed the technical specification limit(Tech. spec. limit is 140% of rated steam flow or 10875 psid.As found condition was 115.0 psid).

Conditions Prior to Occurrence:

Normal operation U-2 at 90%. U-1 at 75%. U-3 Refueling

Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.

Minimum number of switches were operable. (There are four switches per steamline of which two must be operable).

Apparent Cause of Occurrence: Instrument drift.

Analysis of Occurrence:

There was no damage to plant equipment, no injury to personnel, no danger to the health or safety of the public, and no reduction in power generation.

Corrective Action:

Inspected, recalibrated, and functionally tested the switch for repeatability.

Failure Data: 259/7410

*Revision:

*Retention: Period - Lifetime; Responsibility - Administrative Supervisor

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