LICENSEE EVENT REPORT

	CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
01	[A L B R F 2 (2) 0 0 - 0 0 0 0 0 0 0
7 8 CON'T	9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58
01	SOURCE L 6 0 5 0 0 0 2 6 0 7 0 5 2 6 7 9 8 0 6 2 6 7 9 9 9 9 9 9 9 9 9
0 2	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) While in cold shutdown for refueling, local leak rate testing was performed in accor-
0 3	dance to technical specification 4.7.A.2.g. The total leak rate of 5026.5225 SCFH
0 4	exceeded the allowable 60 per cent of LA per 24 hours or 707.1 SCFH. There were no
0 5	significant resulting events nor any danger to health or safety of the public. A pre-
0 6	vious similiar occurrence was reported on BFRO-50-259/7903.
0 7	
08	
7 8 0 9 7 8	SYSTEM CAUSE SUBCODE COMPONENT CODE SUBCODE SU
	LER/RO EVENT YEAR SEQUENTIAL REPORT NO. 17 REPORT 7 9
	ACTION FUTURE EFFECT SHUTDOWN HOURS (22) SUBMITTED FORMSUB. SUPPLIER MANUFACTURER
	$ \begin{array}{c ccccccccccccccccccccccccccccccccccc$
10	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) The excessive total leakage was caused by deterioration of valve seats on the follow-
11	ing systems: Main steam, ventilation and containment dilution, feedwater, RCIC. Valve
1 2	nameplate data is being furnished on a supplement to this report. The valves were
1 3	repaired and retested to demonstrate compliance.
14	
	Solitity Spower Other Status (30) METHOD OF DISCOVERY DESCRIPTION (32) H 28 0 0 0 29 NA B 31 Surveillance Testing
	9 CTIVITY CONTENT 12 13 44 45 46 80 ELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) 10 11 NA 45 45 80
	9 10 11 44 45 80 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 0 0 (37) Z (38) NA
7 8	9 11 12 13 80 PERSONNEL INJURIES 80
1 8	NUMBER DESCRIPTION (41) 0 0 0 0 0 NA 9 11 12
1 9	LOSS OF OR DAMAGE TO FACILITY 43 TYPE DESCRIPTION Z (42) NA
7 8	9 10 80
2 0	SSUED DESCRIPTION (45) NRC USE ONLY NRC USE
-	NAME OF PREPARER PHONE:

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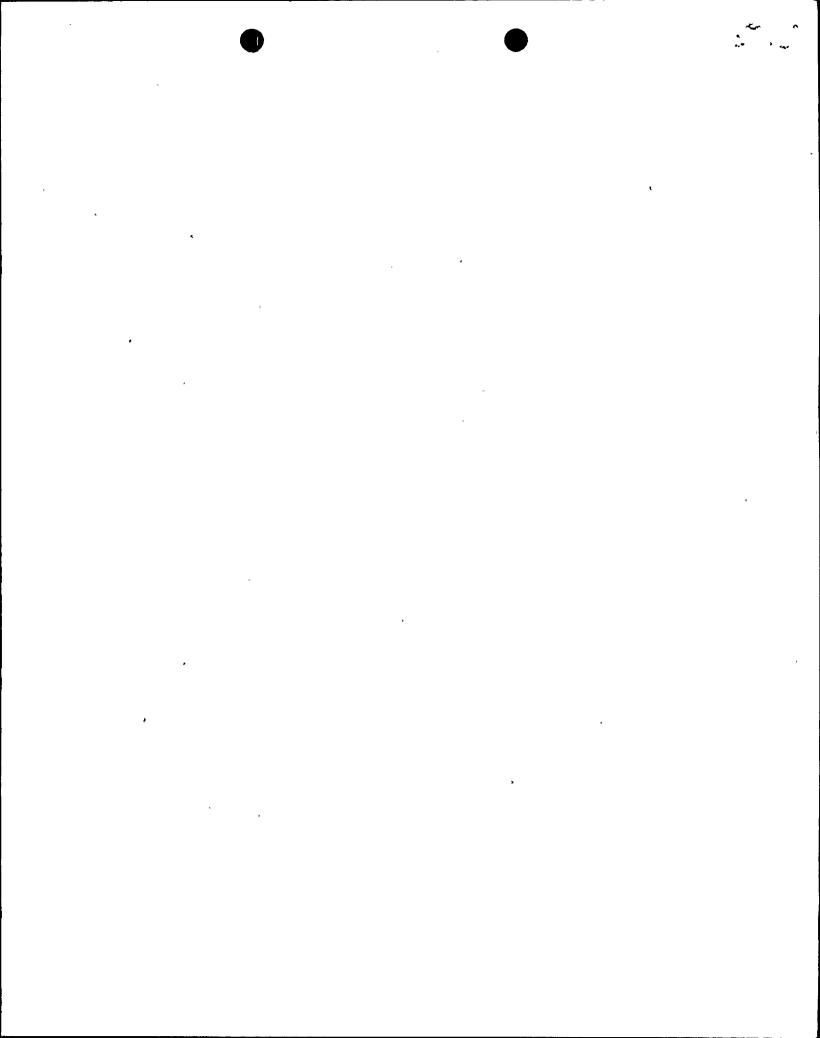
LER SUPPLEMENTAL INFORMATION

BFRO-50- 260 / 7914 Technical Specification Involved 3.7.A .
Reported Under Technical Specification 6.7.2.8.4
Date of Occurrence Unit2
Identification and Description of Occurrence:
The total as found leakage for containment penetrations was found to exceed the allowable 60% LA or 707.1 SCFH. Total leakage was 5026.5225 SCFH.
Conditions Prior to Occurrence:
Ohit 2 shutdown for refueling.
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Action specified in the Technical Specification Surveillance Requirements met due to inoperable equipment. Describe.
None .
Apparent Cause of Occurrence:
There was excessive leakage by valve seating surface.
Analysis of Occurrence:
None
•
Corrective Action:
The seating surfaces were repaired and retested to demonstrate operability. (Refer to the attached lists for valve data and leakage paths).

<u>Failure Data</u>:

BFR0-50-259-793

Tennessee Valley Authority - Browns Ferry Nuclear Plant



VALVE LEAK RATE

PATH	VALVE	VALVE LEAK RATE (SCFH)	PATH LEAK RATE
X-7A	1-14 1-15	206.4968 1846.7659	206.4968
X-7B	1-26 1-27 Ø	454.7919 0.3248	0.3248
X-7D	1-51 1-52	24.8211 342.5316	24.8211
X-205	64,21		3301.7434
X-231	64-29/30/32/33	3,84-19	947.5866
X-25	84-8A/600		264,8690
X-205	84-8.C/603		75,4355
. X-25	84-8D/602		99.7734
х-9в	3-568 71-40 3-572 \Quad 71-39 \Quad \text{69-579 \Quad \text{85-576 \Quad \text{85-576 \Quad \text{85-576 \Quad \text{85-576 \text{85-576 \text{85-576 \text{85-576 \qq	340.2780 317.9633 12,4203 0 3.7809 1.1914	12.4203

 $[\]Omega$ Not reportable - valve leakage given for information to determine path leak rate.

NOTE: 1-14, 1-15, 1-26, 1-51 and 1-52 previously reported for exceeding their allowable leakage rates of 11.5 SCFH.

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VALVE DATA

Valve No.	1-14 1-15 1-26 1-51 1-52	64-21	64–32	3–568	71-40	84–19	84-600 84-602 84-603
MFGR	Atwood & Morrill	Rockwell	Rockwell	Atwood & Morrill	Rockwell	WKM Controls	Kerotest
MATL							
SIZE	26"	20"	18"	24"	6" .	2"	2"
TYPE	Globe	Butterfly	Butterfly	Swing Check	Check	Globe	Check
PRESS. CLASS		e	•			,	
MODEL	20851Н	225498	225498	20788н	PD420688	RF70-28-1- DRT	TV-D-9911S-2
MODE OF OPER.	Air Oper.	Air Oper.	Air Oper.	_	Air Oper.	Air Oper.	
VLV RATING	1250 psig @ 575°F	125#ANS	125#ANS	1700 psig @ 380°F	1500 psig	150 psig	e e e e e
NOP	1150 psig @ 562°F	50# Test	50# Test	•		,	

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