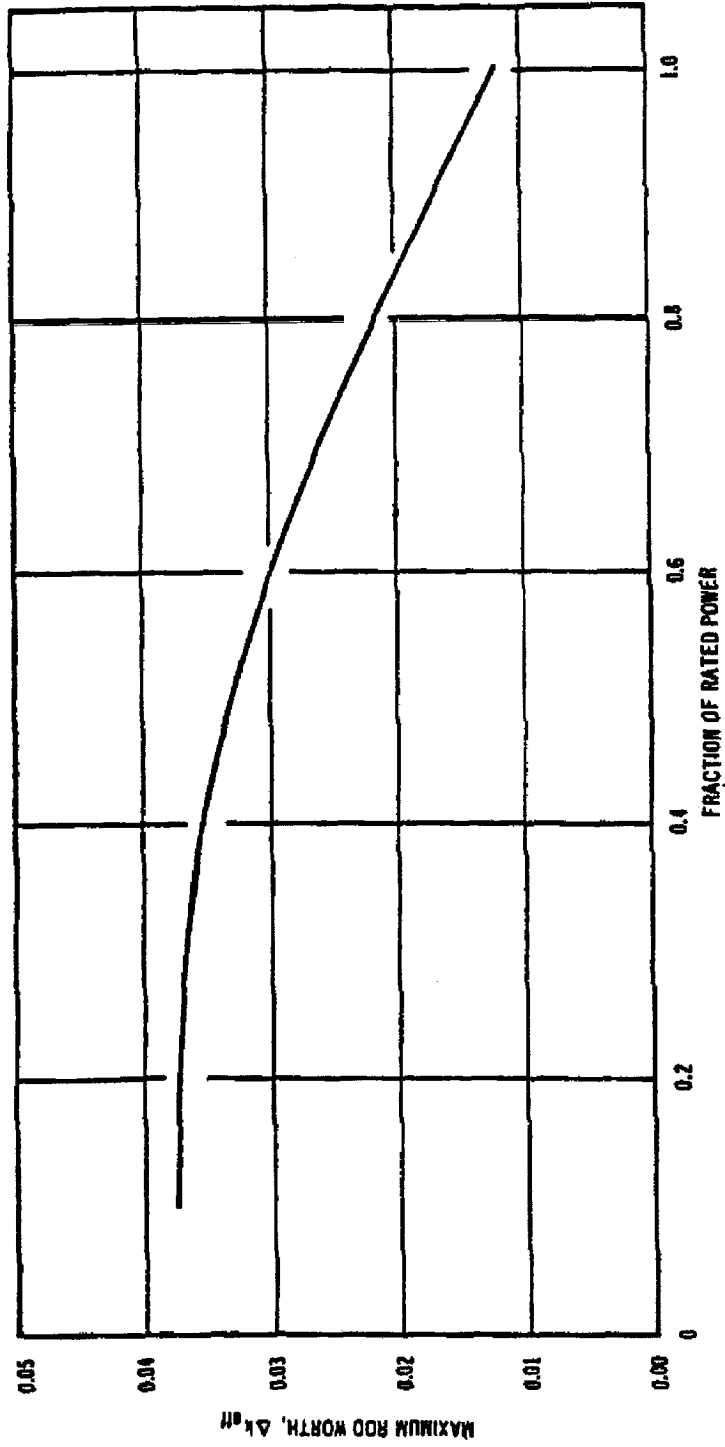


AMENDMENT 17

**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY ANALYSIS REPORT**

Maximum Rod Worth
Versus Moderator Density

FIGURE 14.11-1

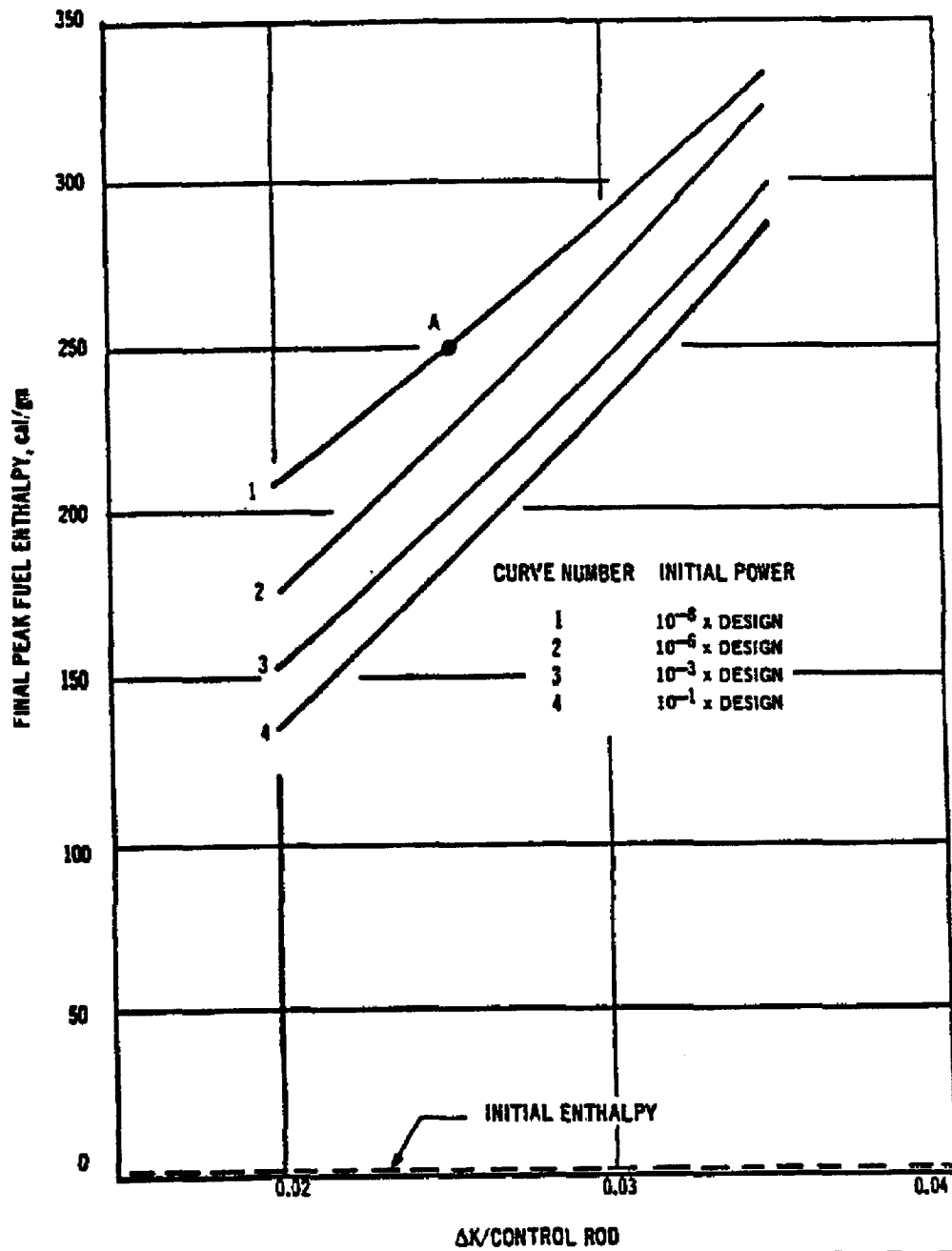


**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY ANALYSIS REPORT**

Maximum Rod Worth
Versus Power Level

FIGURE 14.11-2

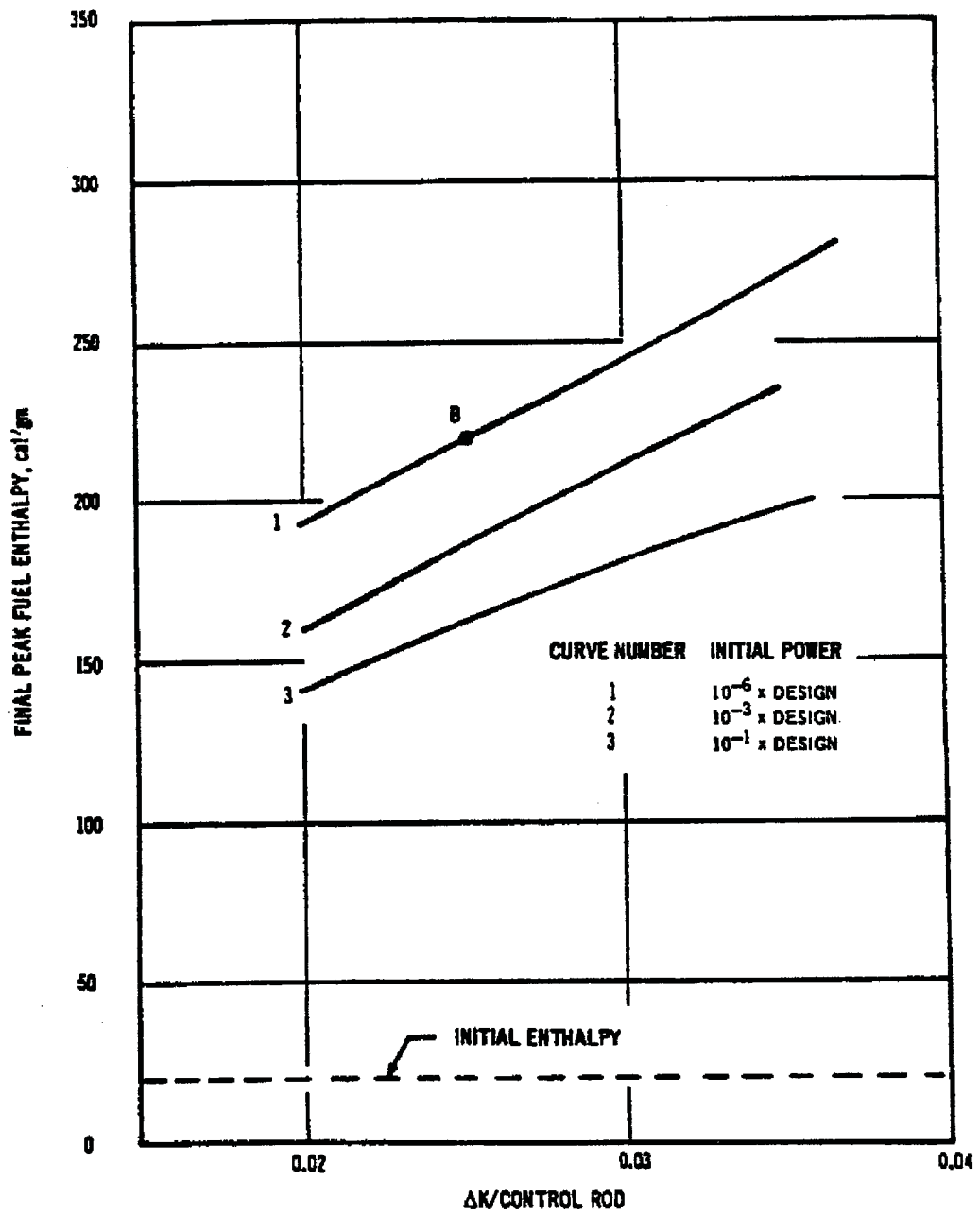
AMENDMENT 17



AMENDMENT 17

BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY ANALYSIS REPORT

Rod Drop Accident (Cold, Critical)
Peak Fuel Enthalpy
FIGURE 14.11-3

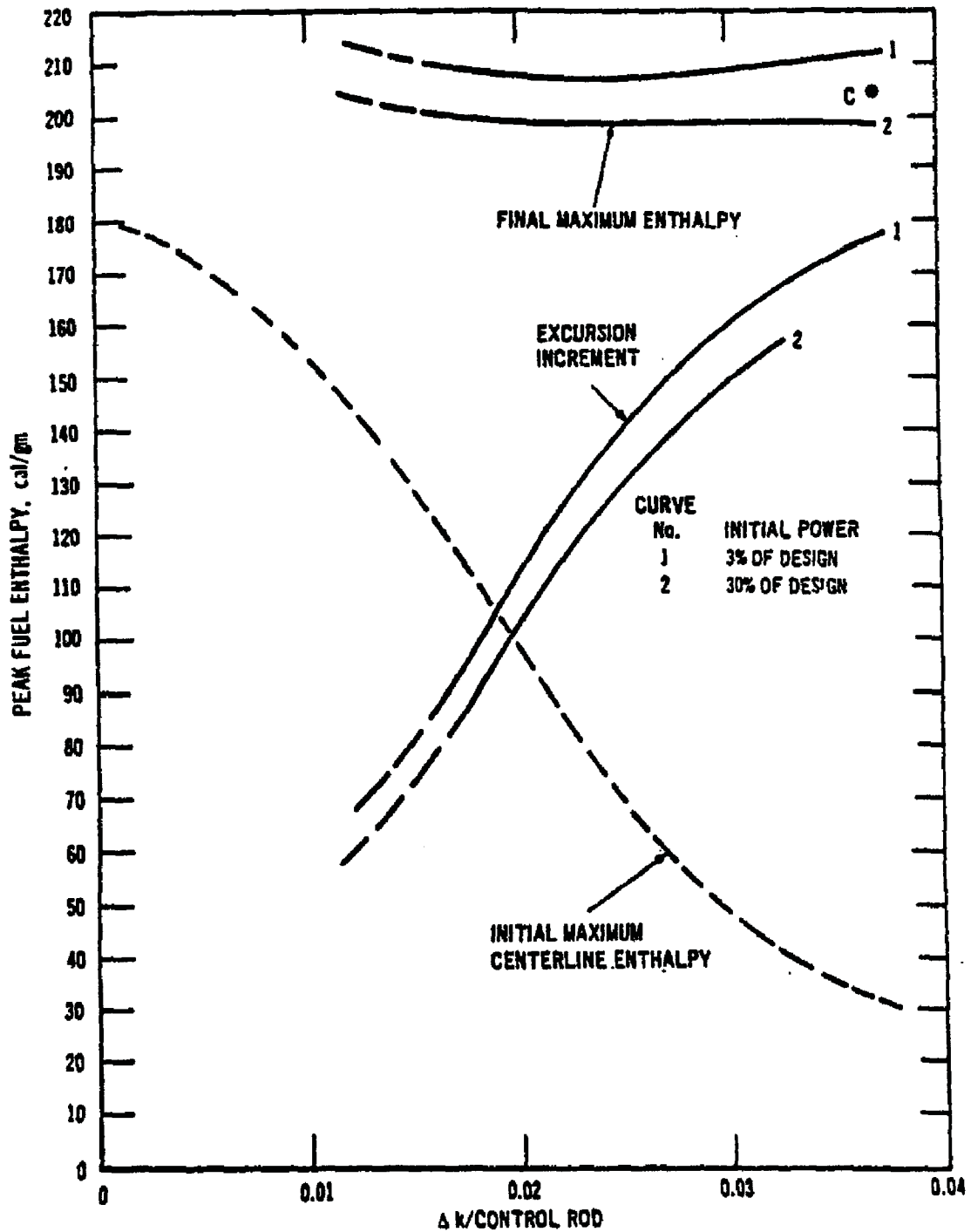


AMENDMENT 17

**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY ANALYSIS REPORT**

**Rod Drop Accident (Hot, Critical)
Peak Fuel Enthalpy**

FIGURE 14.11-4



**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY ANALYSIS REPORT**

Rod Drop Accident (Power Range)
Peak Fuel Enthalpy

FIGURE 14.11-5

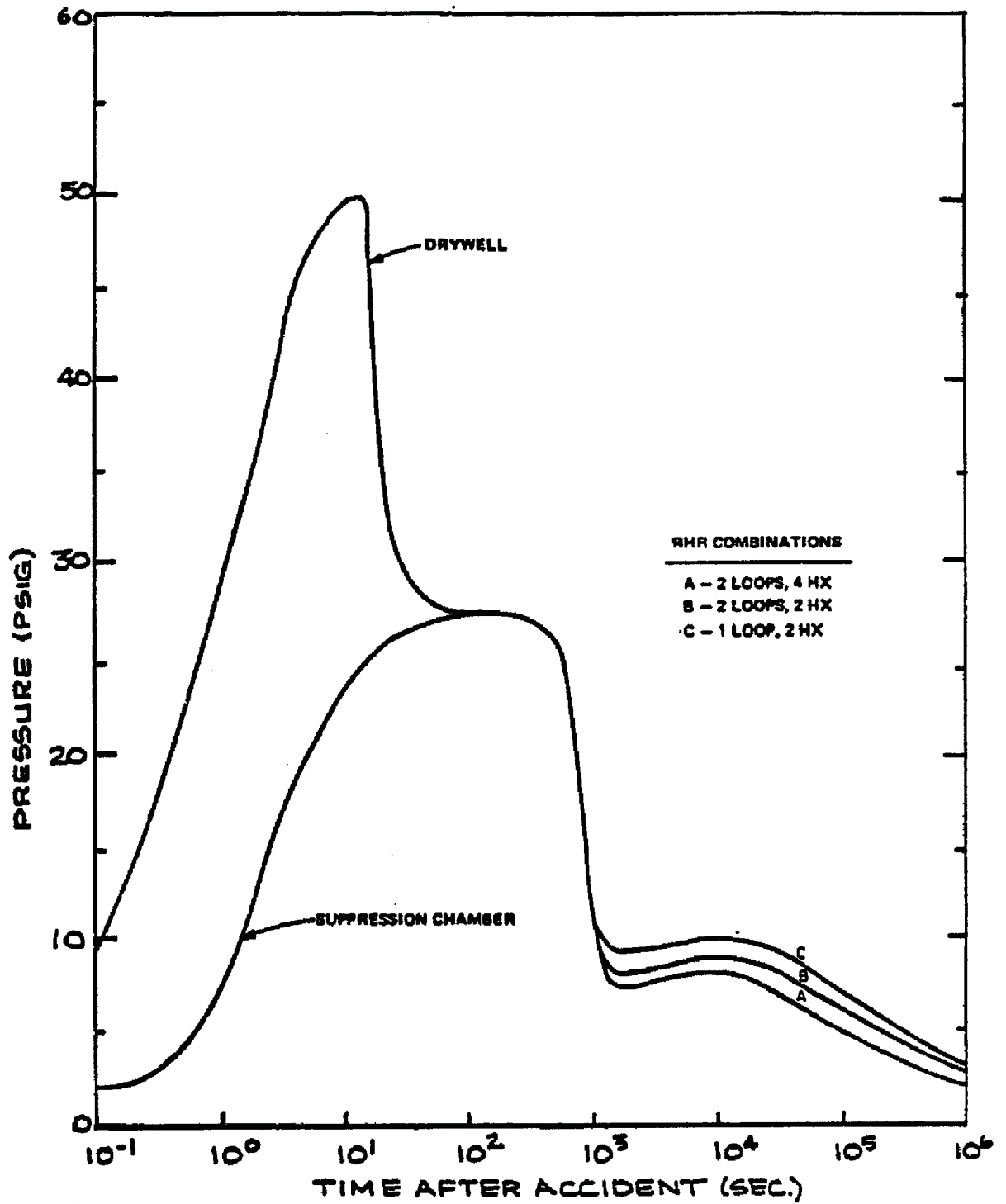
AMENDMENT 17

BFN-17

Figures 14.11-6 through 9

Deleted by Amendment 17

|

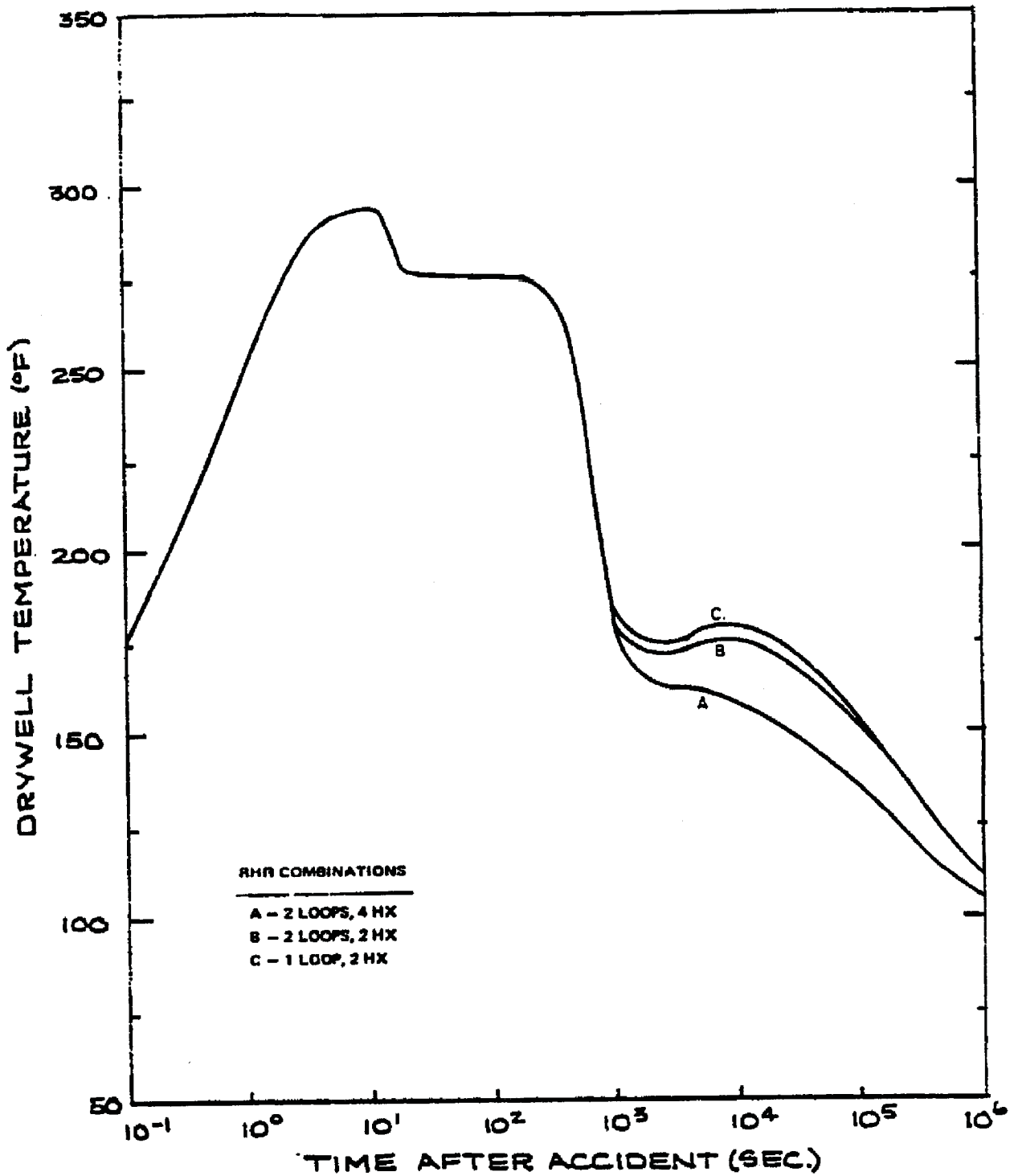


**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY
ANALYSIS REPORT**

**Loss-of-Coolant Accident
Primary Containment Pressure
Response**

FIGURE 14.11-10

AMENDMENT 17

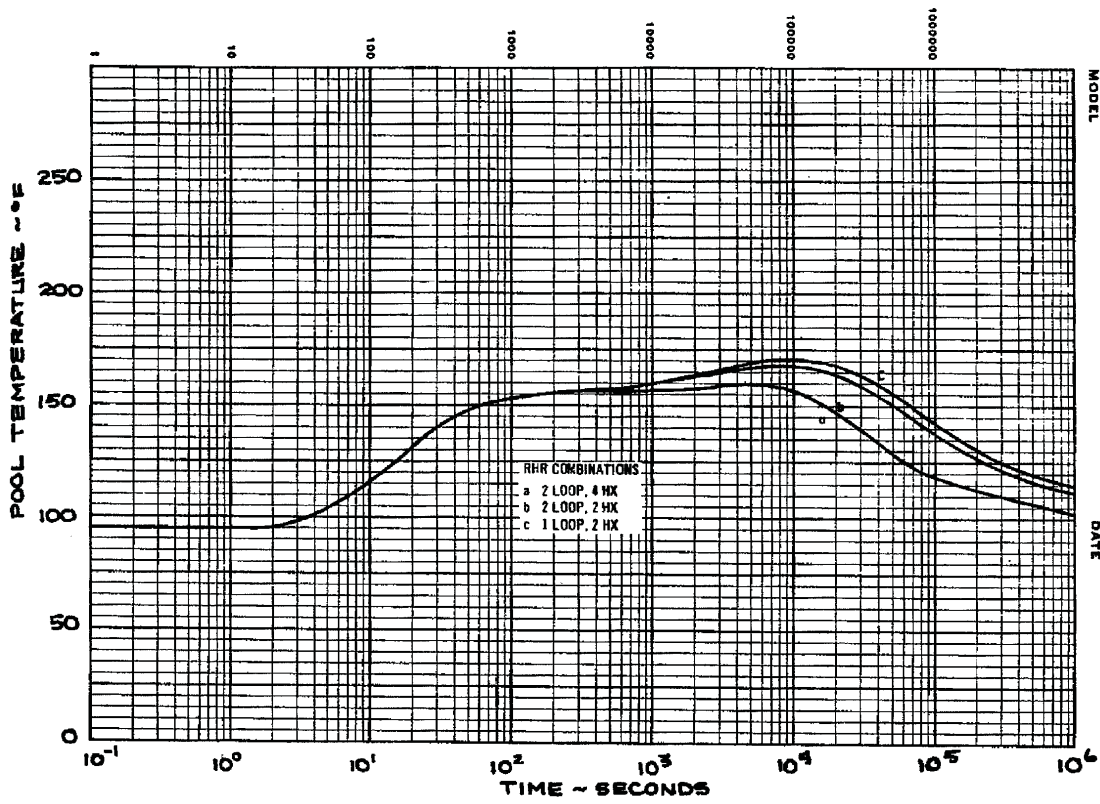


**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY
ANALYSIS REPORT**

Loss-of-Coolant Accident
Drywell Temperature Response

FIGURE 14.11-11

AMENDMENT 17



AMENDMENT 18

BROWNS FERRY NUCLEAR PLANT
 FINAL SAFETY ANALYSIS REPORT

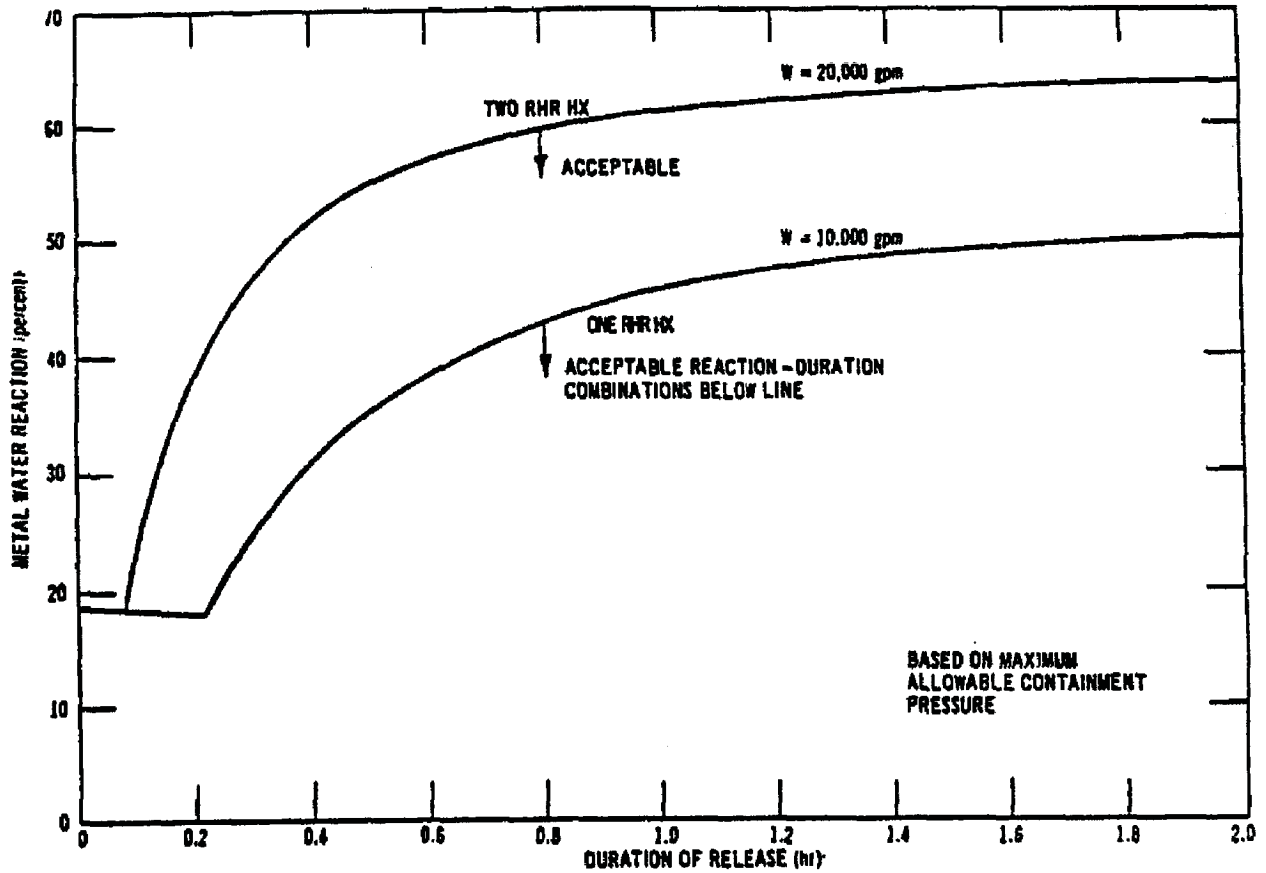
Loss-of-Coolant Accident,
 Pressure Suppression
 Pool Temperature Response
 FIGURE 14.11-12

BFN-17

Figures 14.11-13

Deleted by Amendment 17



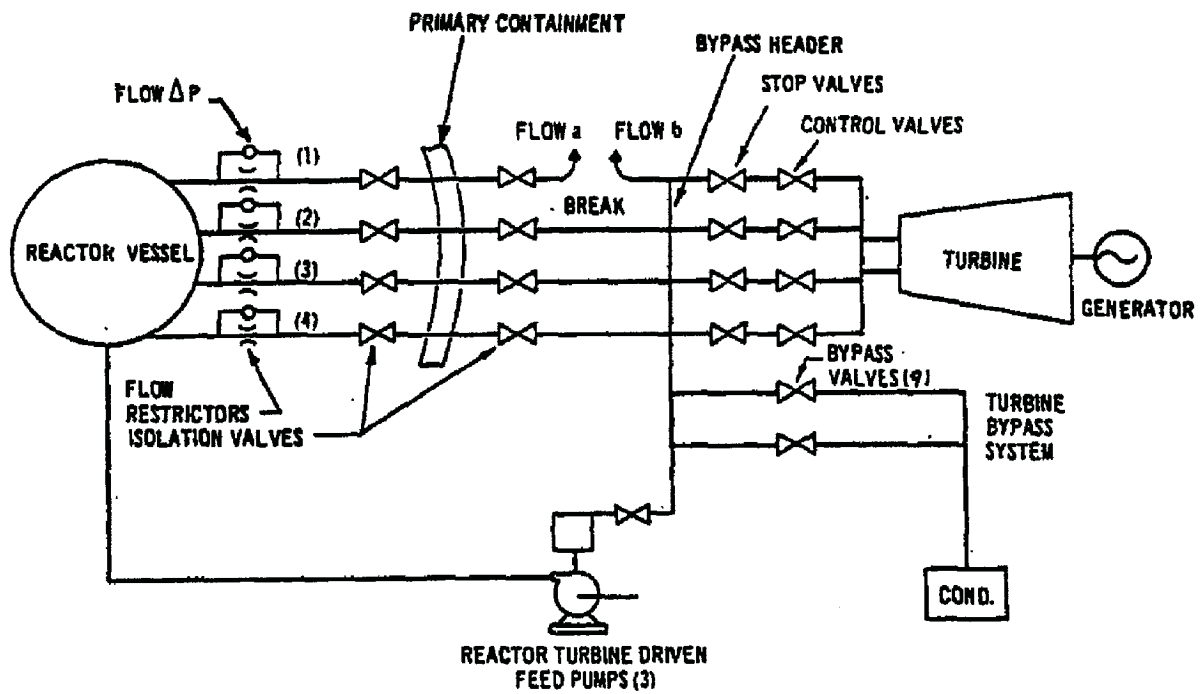


AMENDMENT 17

BROWNS FERRY NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT

Loss-of-Coolant Accident,
Primary Containment Capability
for Metal-Water Reaction

FIGURE 14.11-14

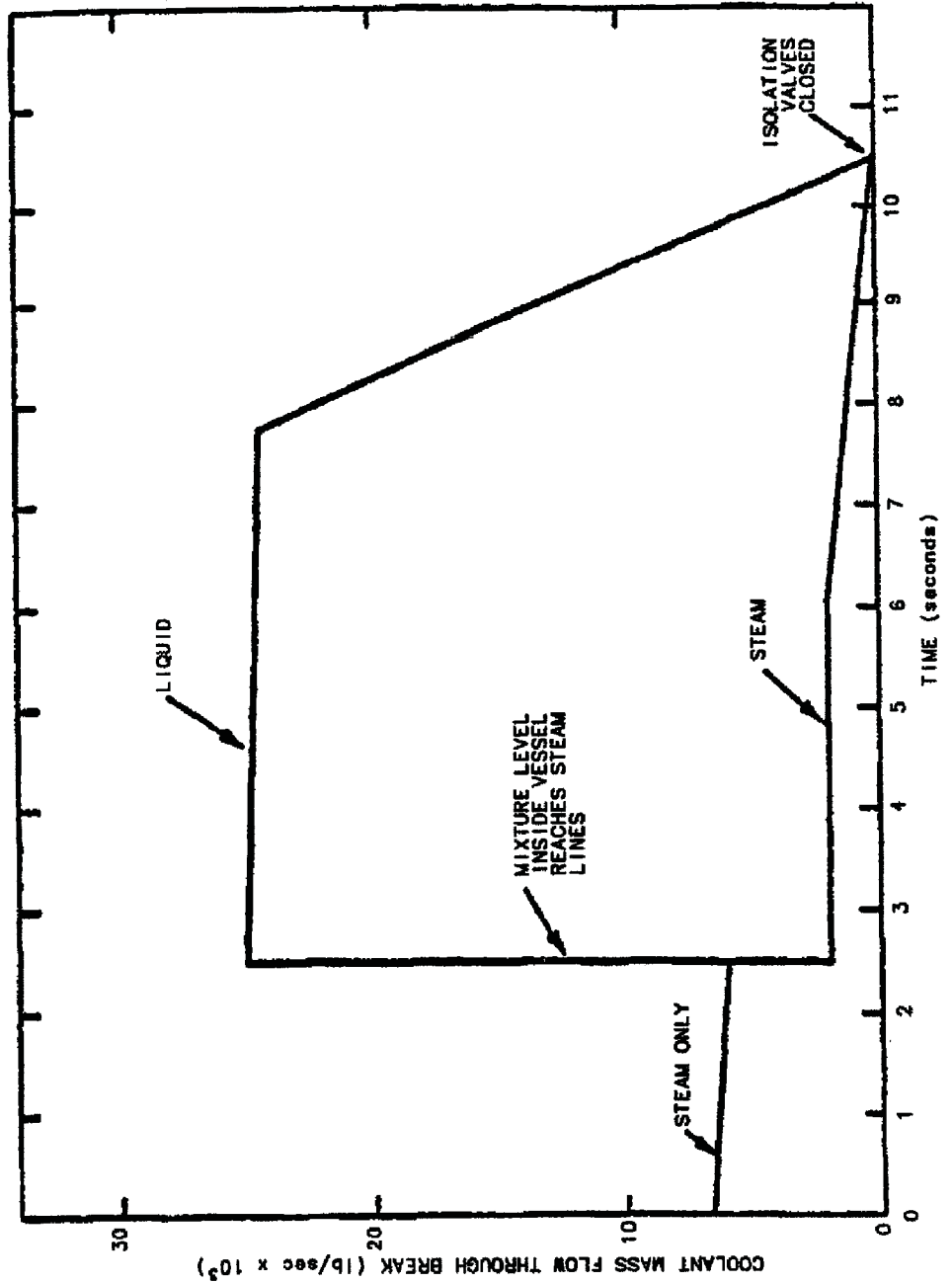


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**BROWNS FERRY NUCLEAR PLANT
FINAL SAFETY
ANALYSIS REPORT**

**Main Steamline Break Accident
Break Location**

FIGURE 14.11-15

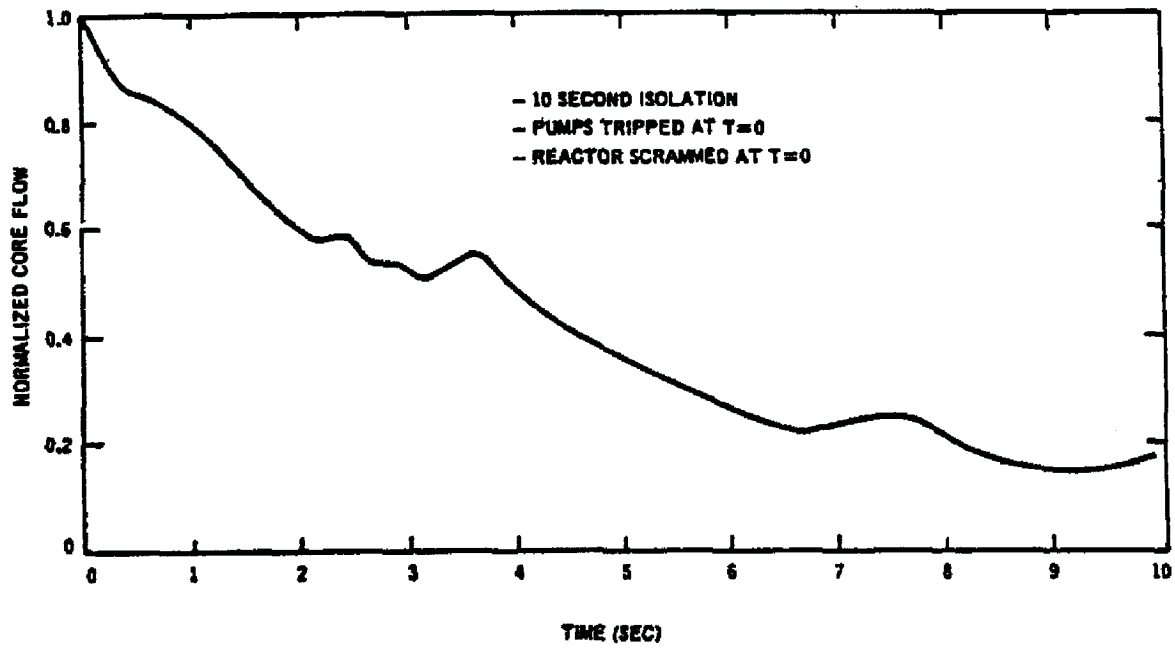


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BROWNS FERRY NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT

MAIN STEAMLINE BREAK ACCIDENT MASS OF COOLANT LOST THROUGH BREAK

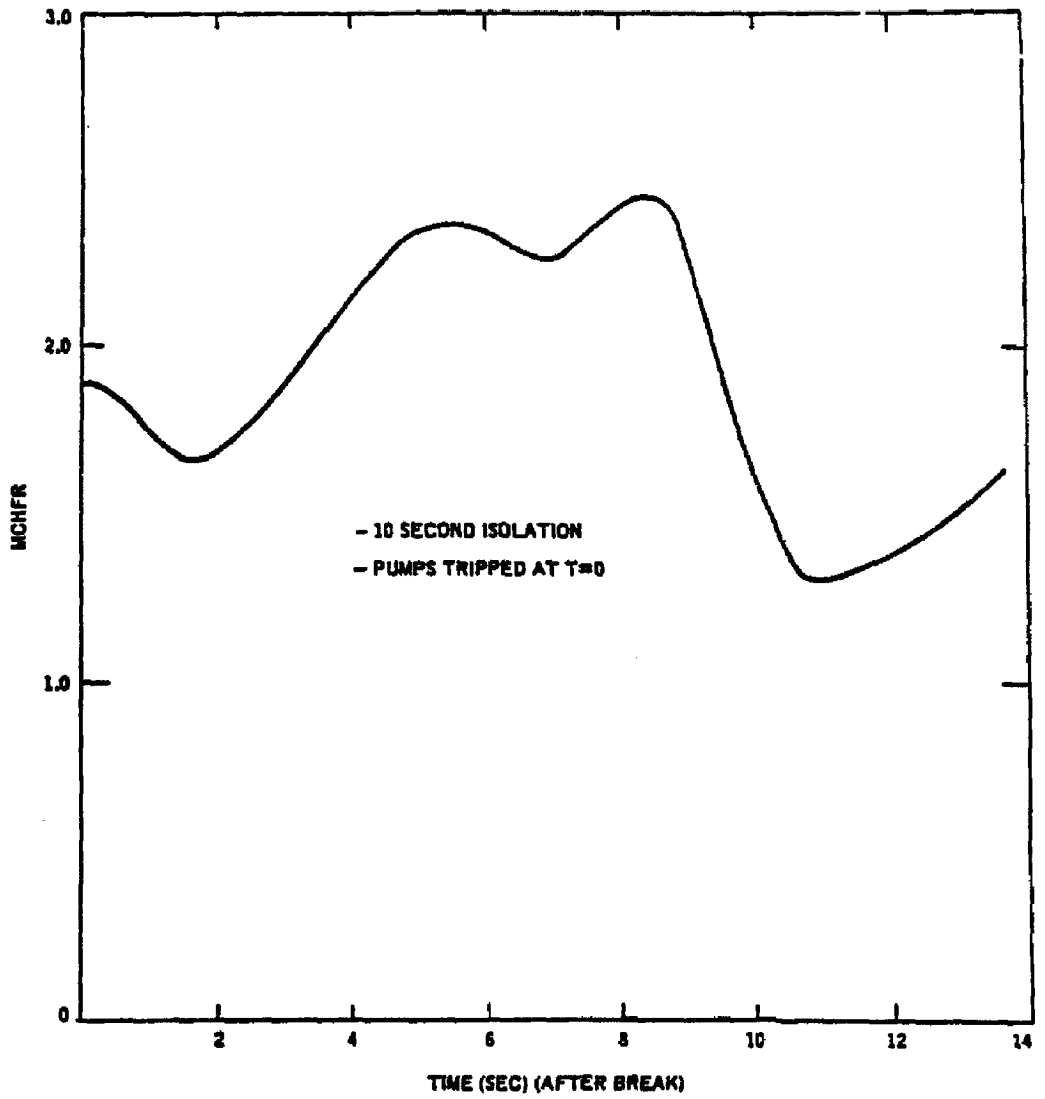
FIGURE 14.11-16



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**BROWNS FERRY NUCLEAR PLANT
 FINAL SAFETY
 ANALYSIS REPORT**

Main Steamline Break Accident
 Normalized Core Inlet Flow
 FIGURE 14.6-17
 FIGURE 14.11-17



AMENDMENT 17

<p align="center"> BROWNS FERRY NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT </p>
<p align="center"> Main Steamline Break Accident Minimum Critical Heat Flux Ratio </p>
<p align="center"> FIGURE 14.11-18 </p>