

C 04/12/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)  
DISTRIBUTION FOR INCOMING MATERIAL

50-307/288

REC: PARR O D  
NRC

ORG: CURTIS N W  
PA PWR & LIGHT

DOCDATE: 04/14/78  
DATE RCVD: 04/19/78

DOCTYPE: LETTER NOTARIZED: NO  
SUBJECT:

COPIES RECEIVED  
LTR 1 ENCL 0

ADVISING ANALYSES OF CHANGES TO OPERATION OF SEVERAL RELIEF VALVES HAVE BEEN COMPLETED, AND THAT LOADS ASSOCIATED WITH SUBSEQUENT SRV ACTUATIONS DO NOT REQUIRE DESIGN MODIFICATIONS FOR REASONS AS LISTED FOR SUBJECT FACILITY..

PLANT NAME: SUSQUEHANNA - UNIT 1  
SUSQUEHANNA - UNIT 2

REVIEWER INITIAL: XJM  
DISTRIBUTER INITIAL: *N*

\*\*\*\*\* DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS \*\*\*\*\*

PSAR/FSAR AMDTS AND RELATED CORESPONDENCE  
(DISTRIBUTION CODE B001)

FOR ACTION: ASST DIR VASSALLO\*\*LTR ONLY(1) BR CHIEF PARR\*\*LTR ONLY(1)  
PROJ MGR MINER\*\*LTR ONLY(1) LIC ASST RUSHBROOK\*\*LTR ONLY(1)

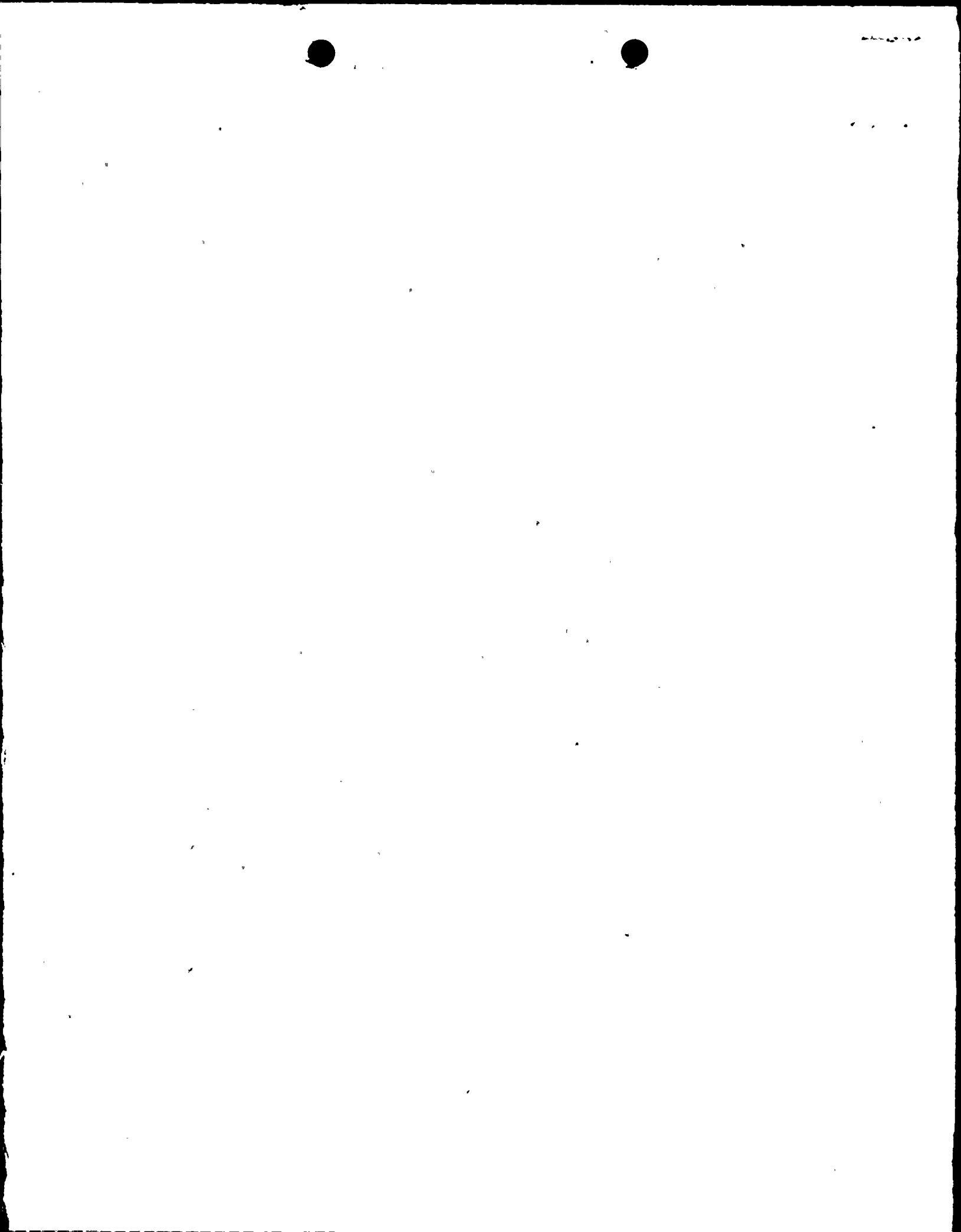
INTERNAL:	REG FILE**LTR ONLY(1)	NRC PDR**LTR ONLY(1)
	I & E**LTR ONLY(2)	OELD**LTR ONLY(1)
	P. COLLINS**LTR ONLY(1)	HOUSTON**LTR ONLY(1)
	HELTEMES**LTR ONLY(1)	CASE**LTR ONLY(1)
	MIPC**LTR ONLY(1)	KNIGHT**LTR ONLY(1)
	BOSNAK**LTR ONLY(1)	SIHWEIL**LTR ONLY(1)
	PAWLICKI**LTR ONLY(2)	ROSS**LTR ONLY(1)
	NOVAK**LTR ONLY(1)	ROSZTOCZY**LTR ONLY(1)
	CHECK**LTR ONLY(1)	TEDESCO**LTR ONLY(1)
	BENAROYA**LTR ONLY(1)	LAINAS**LTR ONLY(1)
	IPPOLITO**LTR ONLY(1)	F. ROSA**LTR ONLY(1)
	GAMMILL**LTR ONLY(4)	VOLLMER**LTR ONLY(1)
	BUNCH**LTR ONLY(1)	J. COLLINS**LTR ONLY(1)
	KREGER**LTR ONLY(1)	KIRKWOOD**LTR ONLY(1)

EXTERNAL: LPDR'S  
WILKES BARRE, PA\*\*LTR ONLY(1)  
TIC\*\*LTR ONLY(1)  
NSIC\*\*LTR ONLY(1)  
ACRS CAT B\*\*LTR ONLY(16)

DISTRIBUTION: LTR 56 ENCL 0  
SIZE: 1P

CONTROL NBR: 781090058  
*MD 4*  
*SD*

\*\*\*\*\* THE END \*\*\*\*\*



APR 14 1978

NORMAN W. CURTIS  
Vice President-Engineering & Construction  
821-5381

cc: Messrs.  
W. E. Barberich N4  
E. M. Mead N5  
A. R. Sabol N4  
Letter File N3  
Date File

Director of Nuclear Reactor Regulation  
Attention: Olan D. Parr, Chief  
Light Water Reactors Branch No. 3  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
CONTAINMENT LOADS FROM RELIEF VALVE OPERATION  
ER 100450 FILES 840-2, 172  
PLA-240

DOCKET NOS. 50-387  
AND 50-388

Dear Mr. Parr:

In our letter to you dated November 17, 1977, we indicated that sufficient analyses had not been completed on the Susquehanna plant to determine if design changes would be necessary due to subsequent operations of several relief valves. These analyses have been completed at this time and we have concluded that the loads associated with subsequent SRV actuations do not require Susquehanna design modifications for the following seasons:

- (1) The techniques (i.e. quenchers) which have been used to enhance suppression pool thermal performance and reduce dynamic pressure effects also mitigate the subsequent actuation effect.
- (2) Test data, which includes subsequent actuation effects, is being used to assess Susquehanna design adequacy.

The preceding items were discussed by Kraftwerk Union during our December 2, 1977 meeting with the NRC staff, and additional supportive information was supplied to you on December 28, 1977. This topic will also be addressed in the Design Assessment Report (DAR) which will be submitted about April 14, 1978.

Very truly yours,

N. W. Curtis  
Vice President-Engineering & Construction

WEB:jm

781090058

RECEIVED DISTRIBUTION SERVICES UNIT

1978 APR 19 PM 2 38

US NRC  
REGULATION SERVICES  
BRANCH

Boo 5/10



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