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 AUTH. NAME AUTHOR AFFILIATION
 CUTTER, A.B. Carolina Power & Light Co.
 RECIP. NAME: RECIPIENT AFFILIATION
 DENTON, H.R. Office of Nuclear Reactor Regulation, Director

SUBJECT: Forwards evaluation outline re proposed resolution to Generic Safety Issue A-43 to demonstrate that containment recirculation sump meets Reg Guide 1.82 provisions. Meeting w/NRC on 850911 requested to discuss analytical methods.

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Carolina Power & Light Company
AUG 29 1985

SERIAL: NLS-85-317

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT
UNIT NO. 1 - DOCKET NO. 50-400
CONTAINMENT RECIRCULATION SUMP

Dear Mr. Denton:

In order to demonstrate that the existing design is a technically sound resolution to the Harris Containment Recirculation Sump Issue--NRC IDI Item D.2.3-1--Carolina Power & Light Company (CP&L) will perform a study using the guidance of the proposed resolution to Generic Safety Issue A-43. Such a study was requested by the NRC Staff in a meeting of August 19, 1985 between the NRC and CP&L.

Carolina Power & Light Company proposes to follow the enclosed evaluation outline derived from the guidance of A-43 to demonstrate the adequacy of the containment sump. We request a meeting with your Staff on September 11, 1985 to discuss the analytical methods being used to resolve this issue.

Although we have agreed to the above study in the interest of resolving an open issue, CP&L believes the current sump design meets all of the provisions of Regulatory Guide 1.82. We believe the NRC Staff has changed its interpretation of what constitutes compliance with this guide based on the Staff's acceptance of our Regulatory Guide 1.82 compliance in the Harris SER dated November 1983. The SER is based upon NRC review of design information presented in Chapter 6 of the Harris FSAR and in responses to Safety Review Question Responses 480.27 and 480.28 dated August 31, 1982 and August 2, 1982, respectively.

If you have any questions, please call.

Yours very truly,

A. B. Cutter - Vice President
Nuclear Engineering & Licensing

ABC/DCM/mf (1885DCM)
Enclosure

cc: Mr. B. C. Buckley (NRC)
Mr. G. F. Maxwell (NRC-SHNPP)
Dr. J. Nelson Grace (NRC-RII)
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Mr. J. L. Milhoan (NRC)

8509040210 850829
PDR ADCK 05000400
A PDR

Street • P. O. Box 1551 • Raleigh, N. C. 27602

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SHEARON HARRIS NUCLEAR POWER PLANT
CONTAINMENT RECIRCULATION SUMP EVALUATION

I. Debris Assessment

- A. Assess areas and volumes of insulation removed by pipe whip, pipe impact, and jet impingement.
- B. Define limiting case for fibrous insulation volume and subsequent transport to sump.
- C. Determine for the limiting case the volume of fibrous insulation transported to the sump screen.

II. Sump Performance

- A. Determine available NPSH.
- B. Compare to required NPSH and identify margin.



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