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SUBJECT: Responds to NRC 850325 request for addl info re responses to Generic Lir 83-28.Reactor trip sys components reviewed & identified as safety-related on component level Q-list.

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MAY 3 1 1985

Mr. Harold R. Denton, Director Office of Nuclear Reactor Regulation United States Nuclear Regulatory Commission Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT UNIT NO. 1 - DOCKET NO. 50-400 GENERIC LETTER 83-28 REQUEST FOR ADDITIONAL INFORMATION RESPONSES

REFERENCE: March 25, 1985 letter from George W. Knighton (NRC) to Mr. E. E. Utley (CP&L)

Dear Mr. Denton:

Carolina Power & Light Company (CP&L) encloses for your review our responses to the request for additional information supplied by the NRC Staff via the referenced letter.

Carolina Power & Light Company considers this information sufficient to resolve this issue. If you have any questions, please contact Mr. Gregg A. Sinders at (919) 836-8168.

Yours very truly,

S. Q. Zimmerman Manager Nuclear Licensing Section

SRZ/GAS/mf (1531GAS)

Enclosure

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PDR

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cc: Mr. B. C. Buckley (NRC) Mr. G. F. Maxwell (NRC-SHNPP) Dr. J. Nelson Grace (NRC-RII) Mr. Travis Payne (KUDZU) Mr. Daniel F. Read (CHANGE/ELP) Mr. D. Lasher (NRC-RRAB) Wake County Public Library Mr. Wells Eddleman Mr. John D. Runkle Dr. Richard D. Wilson Mr. G. O. Bright (ASLB) Dr. J. H. Carpenter (ASLB) Mr. J. L. Kelley (ASLB)

411 Fayetteville Street • P. O. Box 1551 • Raleigh, N. C. 27602



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Item 2.1 (part 1) - Incomplete

Supply a statement confirming that reactor trip system components were reviewed and that they are identified as safety-related on documents, procedures, and information handling systems.

Response

The reactor trip system components were reviewed and they are identified as safety-related on the component level Q-list. This Q-list is used as input for plant documents, procedures and information handling systems. A copy of the reactor protection system and rod control system portions of the Q-list is provided for your information (Attachment 1). The reactor trip and bypass breakers are included on the rod control system portion of the Q-list and are shown as Q-Class "A", or safety-related. Q-Class "E" components are non-safety related. For further information on preparation and control of component level classification, refer to the response to Item 2.2.1.



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1	CPL&L CO-	<u>Attachment 1</u> EQUIPMENT DATA BASE SYSTEM	PAGE 293
3	RUN_DATE 05/14/85	QUALITY LIST	REP EB88-01
3	RUN TIME 13.24.52	DELIVER TO 17C5 HARGETT	
4	UNIT SYSTEM SYST		······································
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11			•
13	DFH	SAFEGUARDS TEST CABINET	
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15	A B	TRAIN B	
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20	DFM	TRIP STATUS LIGHT BOX	
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29	ČS-SL	SURB STEAMLINE ISO. RESET TRAIN B	
30	A CS-10	OBSA STEAMLINE S.I. BLOCK RESET A	1
31	A CS-10	OBSB STEAMLINE S.I. BLOCK RESET B	
32	$\frac{A}{CS-14}$	ISA PRAK PRES. S.I. BLUCK RESET IN-A	
33		5 ISA S. I. RESET TRAIN A MCB	
35	A CS-45	5.2SA S.I. RESET TRAIN A ACP	
36	A CS-45	5.3SB S.I. RESET TRAIN B MCB	-
37	A CS-45	5.458 S.I. RESET TRAIN B ACP	
38	A _ CS-81	OSA F.W. ISO. BYPASS VALVES RESET A	
39		OSB F.W. ISU. BYPASS VALVES RESET	
40 A 1	$\frac{1}{-1}$	SR NAJR SOURCE RANGE BLOCK RESET	
42	A CS-89	. 1SA NJBA INTERMEDIATE RANGE BLOCK	
43	A CS-89	. 1SB N38B INTERMEDIATE RANGE BLOCK	
44	A <u>CS-89</u>	.25AN47A_POWER_RANGE_BLOCK	
45	A CS-89	.25B N47B POWER RANGE BLOCK	1
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47	$A \qquad CS=92$	ISAR SWITCH #1 S.T. ACTORITON	
40	A CS-94	25AB SWITCH #2 S.I. ACTUATION	
50		BREAKER, 120VAC INSTRUMENTATION CIRCUIT	
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3 RL	JN TIME 17.34.53	DELIVER TO CPB 7C5 G. SINDERS	
5	INIT SYSTEM SY	STEM NAME	
6	EQUIPM	ENT CODE EQUIPMENT DESCRIPTION	
7	Q-CLS SPE	CIFIC ID SPECIFIC DESCRIPTION TAG NUMBER	
<u>8</u>	4005 000		
		BREAKER, 4BOV AUX, BUS	
1	E 1D2-	-6D MG SET MOTOR 1A	
2	E1E2 [,]	-2A MG SET MOTOR 1B	<u> </u>
13	DFV	BREAKER, DC DISTRIBUTION PANEL	
	A UP-	1A-5A-18 GENERATUR UUTPUT 1A	
6	F DP-	14-2-18 SERVES ROD DRIVE FWR SUPPLICUE 2	· · · · · · · · · · · · · · · · · · ·
17	A DP-	1B-SB-18 GENERATOR OUTPUT 1B	
18	LES	. INDICATOR, SPEED MCB-1C	
9	E SI-	408 INDICATOR, SPEED MCB-1C	
<u>°</u>	LEU		
	E 102'	-6D SWIICH, MUTUK DISCUNNECT	
3	LEV 162	REGULATOR, VOLTAGE	1
4	E 1A	ROD DRIVE POWER SUPPLY CUB 1	-
5	E 1B	ROD DRIVE POWER SUPPLY CUB 2	
6	LEW	SHUNTS, AM & REV. CURR RELAY, 1200A, 5MV	l l
7	E - 1AA	SHUNTS, AM & REV. CURR RELAY, 1200A, 5MV	
	E 148	SHUNIS, AM & REV. CURR RELAY, 1200A, 5MV	
10	E 1BA	SHUNTS, AM & REV. CURR RELAY, 1200A, 5MV	
11	E 1BB	SHUNTS, AM & REV. CURR RELAY, 1200A, 5MV	
12	_E1BC	SHUNTS, AM & REV. CURR RELAY, 1200A, 5MV	
33	LEX	SWITCH, OVER-TEMP ALARM	
14			ł
16	LEY	SWITCH. REV CURR ALARM	
37	E 1A	ROD DRIVE POWER SUPPLY CUB 1	
8	E '1B	ROD DRIVE POWER SUPPLY CUB 2	
9	LEZ	SWITCH, OVER-VOLT ALARM, ROD DRIVE	
<u>" </u> -	E. 1A F 1B		
12	LFA	SWITCH. MTR BKR TRIP ALARM	
13	E 1A	ROD DRIVE POWER SUPPLY CUB 1	
14 <u> </u>	<u>E' 1B</u>	ROD DRIVE POWER SUPPLY CUB 2	
15	LFC CPA	SWITCH, ROD LIFT DISCONNECT	
10	E CBA	-1 CUNIRUL BANK A MECH 1 -2 CONTROL BANK A MECH 2	
18	E CBA	-3 CONTROL BANK A MECH 3	
49	E CBÁ	-4 CONTROL BANK A MECH 4	
50	E CBA	-5 . CONTROL BANK A MECH 5	•
51	E CBA	-6 CONTROL BANK A MECH 6	
53			
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UN DATE	05/23/85	EQUIP	MENT DATA BASE SYSTEM OUALITY LIST	PAGE 3 REP EB88-01
UN TIME	17.34.53	DELIVER TO CPB 7C5	G. SINDERS PLANT_03	
UNIT SYS	STEM SYSTEM NAME	FOULPMENT DESCRIPTION		•
Q-CLS	SPECIFIC ID	SPECIFIC DESCRIPTION	TAG NUMBER	n (
1 100	65 ROD CONTROL	SYSTEM		•
E	CBA-7	CONTROL BANK A MECH 7		•
<u> </u>	CBA-8	CONTROL BANK A MECH 8		
E	CBB-1 CBB-2	CONTROL BANK B MECH 1		
E	CBB-3	CONTROL BANK B MECH 3		
E	CBB-4	CONTROL BANK B MECH 4		
E	CBB-5	CONTROL BANK B MECH 5		
E	CBB-0 CBB-7	CONTROL BANK B MECH 6		
Ē	CBB-8	CONTROL BANK B MECH 8		
E	CBC-1	CONTROL BANK C MECH 1		- · ·
E	CBC-2	CONTROL BANK C MECH 2		
Ë -	CBC-4	CONTROL BANK C MECH 3		·
E	CBC-5	CONTROL BANK C MECH 5		
E	CBC-6	CONTROL BANK C MECH 6		
F	CBC-8	CONTROL BANK C MECH 7		
Ē	CBD-1	CONTROL BANK D MECH 1		
E	CBD-2	CONTROL BANK D MECH 2		
E	CBD-5	CONTROL BANK D MECH 5		
<u>Е</u>	SBA-1	SHUTDOWN BANK A MECH 1		
Ē	SBA-2	SHUTDOWN BANK A MECH 2		•
E	SBA-3	SHUTDOWN BANK A MECH 3		
<u> </u>	<u>SBA-4</u>	SHUTDOWN BANK A MECH 4		
Ē	SBA-7	SHUTDOWN BANK A MECH 7		
E	SBA-8	SHUTDOWN BANK A MECH 8	•	
<u> </u>	SBA-9	SHUTDOWN BANK A MECH 9	· · · · · · · · · · · · · · · · · · ·	
F	588-1 588-2	SHUTDOWN BANK B MECH 1 SHUTDOWN BANK B MECH 2	a	•
Ē	SBB-3	SHUTDOWN BANK B MECH 3		
E	SBB-4	SHUTDOWN BANK B MECH 4	· · · · · · · · · · · · · · · · · · ·	
E	SBB-6 SBB-7	SHUTDOWN BANK B MECH 6		
E	SBB-8	SHUTDOWN BANK B MECH 8		
E	SBB-9	SHUTDOWN BANK B MECH 9		
E	SBC-1	SHUTDOWN BANK C MECH 1		
E	SBC=2 SBC=3	SHUTDOWN BANK C MECH 2 SHUTDOWN BANK C MECH 3		
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PL&L CO		EQUIPMENT DATA BASE SYSTEM	PAGE 4
UN DATE	05/23/85	QUALITY LIST	REP EB88-01
UN TIME	17.34.53	DELIVER TO CPB 7C5 G. SINDERS	
UNIT SY	STEM SYSTEM NAME		
	EQUIPMENT CODE	EQUIPMENT DESCRIPTION	•
Q-CLS	SPECIFIC ID	SPECIFIC DESCRIPTION TAG NUMBER	•
1 10	65 ROD CONTROL	SYSTEM ·	
_	LFC	SWITCH, ROD LIFT DISCONNECT	
E	SBC-4	SHUTDOWN BANK C MECH 4	
<u>-</u> -	SBD-1	SHUTDOWN BANK D MECH 2	
Ē	SBD-3	SHUTDOWN BANK D MECH 3	
E	SBD-4	SHUTDOWN BANK D MECH 4	
	LFD	FUSE, CHASE SHAWMUT AMP-TRAP, FUSE&VOLT	
E	1	FUSE, CHASE SHAWMUT AMP-TRAP, FUSEAVOLT	
•	LFH	PUSHBUTTON, TRIP TEST	
E	1D2-6D	PUSHBUTTON, TRIP TEST	
E	1E2-2A	PUSHBUTTON, TRIP TEST	
F	102-60		
Ē	1E2-2A	PUSHBUTTON, CLOSE TEST	
	LFL	INDICATOR, TEMPERATURE, MCB	
ε	TI-408A	INDICATOR, TEMPERATURE, MCB	
F	LFM	SWITCH, TEMPERATURE, MCB	
<u> </u>	15-409 I FN	RECORDER, TEMPERATURE, MCB	
Έ	TR-408	· RECORDER, TEMPERATURE, MCB	
	LFO	RECORDER, POSITION	•
<u> </u>	ZR-409	RECORDER, POSITION	
F	1F11-6F	BREAKER WITH STARTER. 480V MCC	
-	LFR	PUSHBUTTON	
E	499-DOWN	PUSHBUTTN, DOWN LCL CNTRL ST.499	
E	499-UP	PUSHBUTTON, UP LCL CNTRL ST.499	
F	LF5 1-151	SWITCH, CUNIKUL SWITCH HOIST HOPED LIMIT	•
Ē	1-LS2	SWITCH, HOIST LOWER LIMIT	
E	1-LS3	SWITCH, GRIPPER ASSEMBLY RELEASE SOLENOI	
-	LFV	MOTOR, ROD DRIVE M-G SET	
F	1A 1R	MUTUR, RUD DRIVE M-G SET	
	LFW	RELAY	<u></u>
E	JS/408	ROD CONTROL	
_	LFX	ROD SPEED CARD	
<u> </u>	SK-408		
Е	JY-408C	ROD CONTROL	
E	JY-408D	ROD CONTROL	
	• •		
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JN DATE 05/23/85	QUALITY LIST	REP EB88-01
JN TIME 17.34.53	DELIVER TO CPB 7C5 G. SINDERS Plant 03	
NIT SYSTEM SYSTEM NAM		
Q-CLS SPECIFIC ID	SPECIFIC DESCRIPTION TAG NUMBER	
1065 ROD CONTRO		······
E JY-408E	ROD CONTROL	•
E JY-408J	ROD CONTROL	
	ROD CONTROL	
E JY/408KP1	ROD CONTROL	
E PY-446	ROD CONTROL	
E QY-479	ROD CONTROL	
E 51-4080 F TD/408B		
E TY-408B	ROD CONTROL	
E TY-408C	ROD CONTROL	
E TY-408F	ROD CONTROL	_
E 11-408G E TY-408H		-
E TY/408HP1	ROD CONTROL	· · · · · · · · · · · · · · · · · · ·
NIG	TRIP, SHUNT	
A BYA	REACTOR SW GEAR BYPASS	• •
	REACTOR SW GEAR BYPASS	
A RTB	REACTOR SW GEAR	
NIH	TRIP, UNDERVOLTAGE	
A BYA	REACTOR SW GEAR BYPASS	
A BYB	REACTOR SW GEAR BYPASS	
	REACTOR SW GEAR	
025	BREAKER, REACTOR TRIP	
A BYA -	REACTOR SW GEAR BYPASS	
A BYA	REACTOR SW GEAR BYPASS	*
A BYB	REACTOR SW GEAR BYPASS DEACTOR SW GEAR BYPASS	
A RTA	REACTOR SW GEAR	_
A RTA	REACTOR SW GEAR	
A RTB	REACTOR SW GEAR	
<u>A RIB</u>	CARIMETS DOWED	
E SCD	SHUTDOWN BK C GRP 1 & BK D GRP 2	
E 1AC	SHUTDOWN BANK A GROUP 1	
E 1BD	SHUTDOWN BANK B GROUP 1	
	SHUTDOWN BANK A GROUP 2 SHUTDOWN BANK B GDOUD 2	
029	MECHANISM, FULL LENGTH ROD CONTROL DRIVE	
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1 2 3	CPL&L RUN D RUN T	CO ATE 05/23/85 IME 17.34.53	EQUIPMENT DATA BASE SYSTEM QUALITY LIST DELIVER TO CPB 7C5 G. SINDERS	PAGE 6 REP EB88-01
4 5 6 7 8	UNIT Q-	SYSTEM SYSTEM NAME Equipment code CLS Specific ID	EQUIPMENT DESCRIPTION SPECIFIC DESCRIPTION TAG NUMBER	· ·
9 10 11 12	1 E E	1065 ROD CONTROL 029 CBA-1 (F-2) CBA-2 (B-10)	SYSTEM MECHANISM, FULL LENGTH ROD CONTROL DRIVE Control Bank A Control Bank A	
13 14 15 16	E	CBA-3 (K-14) CBA-4 (P-6) CBA-5 (K-2) CBA-6 (B-6)	CONTROL BANK A CONTROL BANK A CONTROL BANK A CONTROL BANK A	
17 18 19 20	E	CBA-7 (F-14) CBA-8 (P-10) CBB-1 (F-4) CBB-2 (D-10)	CONTROL BANK A CONTROL BANK A CONTROL BANK B CONTROL BANK B	
21 22 23 24	E	CBB-3 (K-12) CBB-4 (M-6) CBB-5 (K-4) CBB-6 (D-6)	CONTROL BANK B CONTROL BANK B CONTROL BANK B CONTROL BANK B	
25 26 27 28	E E E E	CBB-7 (F-12) CBB-8 (M-10) CBC-1 (D-4) CBC-2 (D-12)	CONTROL BANK B CONTROL BANK B CONTROL BANK C CONTROL BANK C	
29 30 31 32	E	CBC-3 (M-12) CBC-4 (M-4) CBC-5 (H-6) CBC-6 (F-8)	CONTROL BANK C CONTROL BANK C CONTROL BANK C CONTROL BANK C	· · ·
33 34 35 36	E E E	CBC-7 (H-10) CBC-8 (K-8) CBD-1 (H-2) CBD-2 (H-14)	CONTROL BANK C CONTROL BANK C CONTROL BANK D CONTROL BANK D	
37 38 39 40	E E E	CBD-5 (B-8) CBD-6 (P-8) SBA-1 (G-3) SBA-2 (C-9)	CONTROL BANK D CONTROL BANK D SHUTDOWN BANK A SHUTDOWN BANK A	
41 42 43 44	E E E E	SBA-3 (J-13) SBA-4 (N-7) SBA-6 (J-3) SBA-7 (C-7)	SHUTDOWN BANK A SHUTDOWN BANK A SHUTDOWN BANK A SHUTDOWN BANK A	
45 46 47 48	E E E E	SBA-8 (G-13) SBA-9 (N-9) SBB-1 (E-5) SBB-2 (E-11)	SHUTDOWN BANK A SHUTDOWN BANK A SHUTDOWN BANK B SHUTDOWN BANK B	
49 50 51 52	E E E	SBB-3 (L-11) SBB-4 (L-5) SBB-6 (G-7)	SHUTDOWN BANK B Shutdown Bank B Shutdown Bank B	
53 54 55 56	·			
57 58 59 60	-	-	•	,

CPL&L CO RUN DATE 05/23/85 RUN TIME 17.34.53	EQUIPMENT DATA BASE SYSTEM QUALITY LIST DELIVER TO CPB 7C5 PLANT O3	PAGE 7 REP EB88-01
UNIT SYSTEM SYSTEM NAME Equipment code Q-CLS Specific ID	EQUIPMENT DESCRIPTION SPECIFIC DESCRIPTION TAG NUMBER	• •
1 1065 ROD CONTROL 029 E SBB-7 (G-9) E SBB-8 (J-9) E SBB-9 (J-7)	SYSTEM MECHANISM, FULL LENGTH ROD CONTROL DRIVE SHUTDOWN BANK B SHUTDOWN BANK B SHUTDOWN BANK B	
E SBC-1 (E-3) E SBC-2 (C-11) <u>E SBC-3 (L-13)</u> E SBC-4 (N-5)	SHUTDOWN BANK C Shutdown Bank C Shutdown Bank C Shutdown Bank C	
E SBD-1 (F-6) E SBD-2 (F-10) E SBD-3 (K-10) E SBD-4 (K-6)	SHUTDOWN BANK D Shutdown Bank D Shutdown Bank D Shutdown Bank D	
032 E 1A E 1B	SETS, ROD DRIVE M.G. MOTOR GENERATOR SET 1A (SOUTH) MOTOR GENERATOR SET 1B (NORTH)	
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Item 2.1 (part 2) – Incomplete

Submit detailed information describing your vendor interface program for reactor trip system components. Information supplies should state how the program assures that vendor technical information is kept complete, current and controlled throughout the life of the plant and should also indicate how the program will be implemented at Shearon Harris, Unit 1.

Response

The Shearon Harris Nuclear Power Plant (SHNPP) reactor trip system is supplied by Westinghouse (NSSS vendor). CP&L has verified that Westinghouse Technical Bulletins applicable to Generic Letter 83-28 have been received. The recommendation of the Westinghouse Technical Bulletins on reactor trip breakers have been reviewed by CP&L and acted upon. Westinghouse has incorporated return receipt letters into their Technical Bulletins. These return receipt letters are followed up by Westinghouse if the receipt is not returned within a reasonable time. Westinghouse is also providing an updated list of Technical Bulletins which CP&L will use as an additional tool in ensuring that applicable Technical Bulletins have been received. Westinghouse has issued a complete list of past recommendations on NSSS equipment. The index of recommendations is also updated periodically to ensure the plant is aware of issued recommendations.

Once the Technical Bulletins are received, the vendor recommendations are processed in accordance with our procedures dealing with Operational Experience Feedback. Vendor recommendations concerning plant equipment are tracked by the Onsite Nuclear Safety Unit in accordance with the above Operating Experience Feedback Program procedures. This program ensures that the information is provided to Operations or Maintenance or other appropriate organizations for their use. The recommendations are tracked until final disposition occurs.

SHNPP has implemented procedures for initial review and revisions to the Technical Manual. Once the Technical Manual has been accepted for use by the Technical Support Unit, Document Control distributes the vendor manuals per Document Control Instruction.

At SHNPP, no distinction is made for on-site processing of vendor information for the reactor trip system components and other safety-related equipment. The current established controls provide for processing of vendor information with the same methodology established for Technical Manuals and Bulletins received.

In summary, CP&L believes the current vendor interface program for the reactor trip system components in conjunction with established plant procedures for maintenance, surveillance testing, equipment repair and replacement, and quality assurance program provide a comprehensive equipment reliability program throughout the life of the plant.



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Item 2.2.1 – Incomplete

Submit information on how equipment will be classified as safety-related and will be designated as such on plant documentation as requested in sub-items 2.2.1.1 to 2.2.1.6.

Response

An established procedure exists at SHNPP for the preparation and control of a component level classification. By procedure, the quality classification of components for quality purposes is obtained by applying the quality designation inherent in existing controlled design documents such as flow diagrams, controlled wiring diagrams, etc. Safety-related components are classified as Q-Class "A" with non-safety/seismic, radwaste, fire protection, and non-safety classified as "B", "C", "D", and "E", respectively. A component may meet more than one quality classification criteria; but in all cases, the highest quality class is the resultant quality classification.

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Item 2.2.2 – Incomplete

Submit detailed information describing your vendor interface program for all safety-related components. Information supplied should state how the program assures that vendor technical information is kept complete, current, and controlled throughout the life of the plant and should also indicate how the program will be implemented at Shearon Harris, Unit 1. If the recommendations of NUTAC are to be implemented at Shearon Harris, Unit 1, you need to supplement your response to address to concern about establishing and maintaining an interface with all vendors of safety-related equipment since the staff found NUTAC lacking in this respect.

Response

A unique aspect about SHNPP construction is the program of using permanent plant staff to order the initial loading of spare parts instead of depending on parts received only with initial equipment purchase. To support this program, a parallel effort is in process to systematically review all vendor technical manuals, starting with safety related vendors first. In this process, each vendor is contacted to verify the information is current, and maintenance personnel perform a review for adequacy of content. Once these technical reviews are completed, the manuals are accepted by Technical Support and sent to Document Control for distribution. As a result of these programs, SHNPP will be assured of having the latest technical manuals for safety-related equipment at time of fuel load.

With regard to continued maintenance of technical manuals, SHNPP intends to retain the function of technical manual upgrade in concert with our Operational Experience Feedback program, consistent with the NUTAC recommendations.

CP&L has reviewed the NUTAC Vendor Equipment Technical Information Program (VETIP) and believes that the NUTAC effort provides effective guidelines in establishing a vendor interface program. The VETIP as defined in the March 1984 NUTAC document is considered a valid response to Section 2.2.2 of the NRC Generic Letter 83-28. CP&L is in the process of implementing a program to meet the intent and guidelines of the NUTAC/VETIP.

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Item 3.1.3 – Incomplete

Results of review of test and maintenance programs shall identify any postmaintenance testing that may degrade rather than enhance safety and shall describe actions to be taken including submitting needed Technical Specification changes.

Item 3.2.3 – Incomplete

Same at Item 3.1.3.

Response

CP&L submitted the "pen and ink" version of the SHNPP Technical Specifications to the NRC on April 23, 1985. These Technical Specifications will undergo a detailed review by CP&L to assess the implementation of the required surveillances. This review will determine if the surveillances proposed by the SHNPP Technical Specifications (which are based upon NUREG-0452, Rev. 5, Westinghouse Standard Technical Specifications) can be implemented. Implicit in this assessment is that: (1) the surveillance does not damage the component, and (2) the applicable surveillance test(s) is/are required prior to declaring a component OPERABLE following maintenance which removes the component from service. The results of this review will be provided to the NRC as changes to the Technical Specifications during the review process for the SHNPP Technical Specifications. However, a specific submittal on the 3.1.3 and 3.2.3 items will not be made.

The Westinghouse Maintenance Manual for DS-416 reactor trip breakers has been received and reviewed. The appropriate maintenance procedures that implement the recommended maintenance action have been drafted.

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Item 4.5.2 - Incomplete

Provide a description of the design provisions that will permit on-line testing of the reactor trip system.

Response

Refer to FSAR Section 7.2.2.3.10 (Revision 17) for a description of the design provisions that will permit on-line testing.

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Item 4.5.3 – Incomplete

Provide results of review of existing or proposed intervals for on-line testing considering the concerns of 4.5.3.1 to 4.5.3.5 in the generic letter. Proposed Technical Specification changes resulting from this review shall be submitted for review.

Response

CP&L's November 7, 1983 submittal on Generic Letter 83-28 responded to this item. CP&L referenced WCAP-10271 "Evaluation of Surveillance Frequencies and Out of Service Times for Reactor Protection Instrumentation Systems" for reduced surveillance intervals for on-line testing of Reactor Protection System Channels. This WCAP was approved by the NRC on February 21, 1985. CP&L's April 23, 1985 Technical Specification submittal incorporates the reduced surveillance intervals within the limitations of the NRC's SER.

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