E 03/10/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-220

REC: GRIER B H
NRC

ORG: SCHNEIDER R R
NIAGARA MOHAWK PWR

DOCDATE: 03/01/78
DATE RCVD: 03/07/78

DOCTYPE: LETTER NOTARIZED: NO

COPIES RECEIVED
LTR 0 ENCL 1

SUBJECT:
LICENSEE EVENT REPT (RO 50-220/78-07) ON 02/06/78 CONCERNING DURING STEADY STATE OPERATAION, AN INSPEC OF THE LIQUID WASTE SURGE TANK FOUND SEVERAL PINHOLE LEAKS...W/ATT LERS 78-008, 78-009 AND 78-010.

PLANT NAME: NINE MILE PT - UNIT 1

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: DL

********** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS **********

INCIDENT REPORTS
(DISTRIBUTION CODE A002)

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NOVAK/CHECK**W/ENCL
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EISENHUT**W/ENCL
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K SEYFRIT/IE**W/ENCL
NRC PDR**W/ENCL
MIPC**W/3 ENCL
HOUSTON**W/ENCL
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BAER**W/ENCL
VOLLMER/BUNCH**W/ENCL
ROSA**W/ENCL

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OSWEGO, NY**W/ENCL
TIC**W/ENCL
NSIC**W/ENCL
ACRS CAT B**W/16 ENCL

DISTRIBUTION: LTR 45 ENCL 45
SIZE: 1P+1P+4P

CONTROL NBR: 780680045

********** THE END **********

REGULATORY DOCKET FILE COPY

NMP-0130

NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

March 1, 1978

RECEIVED MAR 7, 1978 U.S. NUCLEAR REGULATORY COMMISSION Mail Section

Mr. Boyce H. Grier
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

RE: Docket No. 50-220

Dear Mr. Grier:

In accordance with Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit the following Licensee Event Reports:

LER 78-07, which is in violation of Section 6.9.2b(4) of the Technical Specifications
LER 78-08, which is in violation of Section 3.6.2 of the Technical Specifications
LER 78-09, in accordance with the Technical Specifications
LER 78-10, which is in violation of Section 3.6.2 of the Technical Specifications

These reports were completed in the format designated in NUREG-0161, dated July 1977.

Very truly yours,

J.A. Lampages for

R.R. Schneider
Vice President -
Electric Production

mtm

Attachments (3 copies)

xc: Director, Office of I&E (30 copies)
Director, Office of MIPC (3 copies)

780680045

A002/S *
0/1

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | N Y N M P 1 ② | 0 0 - 0 0 0 0 0 - 0 0 ③ | 4 1 1 1 1 ④ | ⑤
LICENSEE CODE: N Y N M P 1
LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0
LICENSE TYPE: 4 1 1 1 1
CAT:

CON'T
0 1 | REPORT SOURCE: L ⑥ | DOCKET NUMBER: 0 5 0 0 0 2 2 0 ⑦ | EVENT DATE: 0 2 0 6 7 8 ⑧ | REPORT DATE: 0 2 2 8 7 8 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 During steady state operation, an inspection of the liquid waste surge
0 3 tank found several pinhole leaks. There were no measurable losses of
0 4 liquid but leaks could be identified by dried deposit tracks on the
0 5 side of the tank. This is reportable per T.S. 6.9.2b(4). This
0 6 condition resulted in minimal safety implications.
0 7
0 8

0 9
SYSTEM CODE: M A ⑪
CAUSE CODE: B ⑫
CAUSE SUBCODE: C ⑬
COMPONENT CODE: A C C U M U ⑭
COMP. SUBCODE: Z ⑮
VALVE SUBCODE: Z ⑯

⑰ LER/RO REPORT NUMBER
EVENT YEAR: 7 8
—
SEQUENTIAL REPORT NO.: 0 0 7
/
OCCURRENCE CODE: 0 3
REPORT TYPE: L
—
REVISION NO.: 0

ACTION TAKEN: X ⑱
FUTURE ACTION: F ⑲
EFFECT ON PLANT: Z ⑳
SHUTDOWN METHOD: Z ㉑
HOURS ㉒: 0 0 0 0
ATTACHMENT SUBMITTED: N ㉓
NPRD-4 FORM SUB.: N ㉔
PRIME COMP. SUPPLIER: N ㉕
COMPONENT MANUFACTURER: P 2 7 0 ㉖

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 A 50,000 gal. aluminum B-209 tank built by Pittsburg-Desmoines Steel Co.
1 1 was constructed with walls 0.1875 inches thick. During assembly, weld-
1 2 ing and grinding reduced the thickness in spots to half of this value
1 3 apparently leaving pinhole leaks. A tank liner is being considered as
1 4 a permanent solution, currently usage is strictly controlled.

1 5
FACILITY STATUS: E ㉘
% POWER: 0 9 9 ㉙
OTHER STATUS ㉚: NA
METHOD OF DISCOVERY: C ㉛
DISCOVERY DESCRIPTION ㉜: Tank Inspection

1 6
ACTIVITY RELEASED: Z ㉝
CONTENT OF RELEASE: Z ㉞
AMOUNT OF ACTIVITY ㉟: NA
LOCATION OF RELEASE ㊱: NA

PERSONNEL EXPOSURES
1 7
NUMBER: 0 0 0 ㊲
TYPE: Z ㊳
DESCRIPTION ㊴: NA

PERSONNEL INJURIES
1 8
NUMBER: 0 0 0 ㊵
DESCRIPTION ㊶: NA

LOSS OF OR DAMAGE TO FACILITY ㊸
1 9
TYPE: Z ㊷
DESCRIPTION: NA

PUBLICITY
2 0
ISSUED: N ㊹
DESCRIPTION ㊺: NA

NRC USE ONLY

NAME OF PREPARER: Dennis K. MacVittie
PHONE: 315-343-2110 ext. 1558

GPO 917-926

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | NYNMP1 (2) | 00-00000-00 (3) | 41111 (4) | (5)
LICENSEE CODE — LICENSE NUMBER — LICENSE TYPE — CAT

CON'T
01 | REPORT SOURCE L (6) | 05000220 (7) | 021078 (8) | 022878 (9)
DOCKET NUMBER — EVENT DATE — REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During steady state operation, performance of surveillance test N1-ISP-
03 RE22, found RE22C with a setpoint of 103.5 PSID, RE22E at 103.0 PSID,
04 and RE22F at 103.5 PSID. Required setpoint for these main steam line
05 high flow indicating switches is 105 PSID +/- 1 PSID. This condition
06 resulted in minimal safety implications. The three instruments drifted
07 in a more conservative direction.
08

09 SYSTEM CODE IB (11) | CAUSE CODE E (12) | CAUSE SUBCODE E (13) | COMPONENT CODE INSTRU (14) | COMP. SUBCODE S (15) | VALVE SUBCODE Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR 78 | — | SEQUENTIAL REPORT NO. 008 | / | OCCURRENCE CODE 03 | REPORT TYPE L | — | REVISION NO. 0

ACTION TAKEN E (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS 0000 (22) | ATTACHMENT SUBMITTED N (23) | NPRD-4 FORM SUB. Y (24) | PRIME COMP. SUPPLIER N (25) | COMPONENT MANUFACTURER B080 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The deviation in setpoints experienced on the Barton Model 278 Differen-
11 tial Pressure Indicating Switches was due to instrument drift. RE22C
12 was reset at 105.5 PSID, RE22E at 105.0 PSID, and RE22F at 104.7 PSID.
13 Current surveillance testing schedules are adequate to insure early
14 detection of future instrument drifts.

15 FACILITY STATUS E (28) | % POWER 099 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY B (31) | DISCOVERY DESCRIPTION (32) Surveillance Testing

16 ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

17 PERSONNEL EXPOSURES NUMBER 000 (37) | TYPE Z (38) | DESCRIPTION (39) NA

18 PERSONNEL INJURIES NUMBER 000 (40) | DESCRIPTION (41) NA

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION (43) NA

20 PUBLICITY ISSUED N (44) | DESCRIPTION (45) NA

NRC USE ONLY

NAME OF PREPARER Dennis K. MacVittie PHONE: 315-343-2110 ext 1558

GPO 917-926

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	N Y N M P 1	②	0 0 - 0 0 0 0 0 - 0 0	③	4 1 1 1 1	④		⑤		
7 8	9 LICENSEE CODE 14		15 LICENSE NUMBER 25		26 LICENSE TYPE 30		57 CAT 58			

CON'T 0 1	REPORT SOURCE L	⑥	0 5 0 0 0 2 2 0	⑦	0 2 2 1 7 8	⑧	0 2 2 8 7 8	⑨
7 8	60		61 DOCKET NUMBER 68		69 EVENT DATE 74		75 REPORT DATE 80	

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 During routine station operation with the TIP detectors withdrawn, ex-
0 3 plosive charges in the TIP shear valves were replaced with new charges.
0 4 Following replacement, it was found that the ohmmeter continuity test
0 5 for acceptable electric contact was unsatisfactory. The normal TIP tube
0 6 isolation system was operable at this time. An evaluation of the safety
0 7 implications is being conducted.
0 8

	SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE		COMP. SUBCODE		VALVE SUBCODE	
0 9	X X	⑪	B	⑫	A	⑬	X X X X X X	⑭	Z	⑮	Z	⑯
7 8	9 10		11		12		13 18		19		20	

⑰ LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	7 8	—	0 0 9	/	0 1	T	—	0
	21 22	23	24 26	27	28 29	30	31	32

ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB.		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER	
F	⑱	Z	⑲	Z	⑳	Z	㉑	0 0 0 0	㉒	Y	㉓	N	㉔	N	㉕	G 0 8 0	㉖
33		34		35		36		37 40		41		42		43		44 47	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 It was found that the actual electrical connections as originally wired
1 1 were not as described in the Maintenance Procedure or in a re-
1 2 vised GE drawing. The wiring was changed to agree with the revised GE
1 3 dwg and all four of the old squib devices were detonated successfully.
1 4

	FACILITY STATUS		% POWER		OTHER STATUS	㉚	METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	㉜
1 5	E	㉘	0 9 9	㉙	NA		B	㉛	Surveillance Testing	
7 8	9		10 12		13 44		45		46 80	

	ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY	㉟	LOCATION OF RELEASE	㊱
1 6	Z	㉝	Z	㉞	NA		NA	
7 8	9		10		11 44		45 80	

PERSONNEL EXPOSURES

	NUMBER		TYPE		DESCRIPTION	㊴
1 7	0 0 0	㊲	Z	㊳	NA	
7 8	9 11		12		13 80	

PERSONNEL INJURIES

	NUMBER		DESCRIPTION	㊶
1 8	0 0 0	㊵	NA	
7 8	9 11		12 80	

LOSS OF OR DAMAGE TO FACILITY ㊸

	TYPE		DESCRIPTION
1 9	Z	㊷	NA
7 8	9		10 80

PUBLICITY

	ISSUED		DESCRIPTION	㊺	NRC USE ONLY
2 0	N	㊹	NA		
7 8	9		10 68		69 80

NAME OF PREPARER D.K. MacVittie PHONE: 315-343-2110 x 1558

GPO 917-926

NIAGARA MOHAWK POWER CORPORATION/300 ERIE BOULEVARD WEST, SYRACUSE, N.Y. 13202/TELEPHONE (315) 474-1511

February 28, 1978

Mr. Boyce H. Grier
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

RE: Docket No. 50-220
LER 78-09/01T-0
Nine Mile Point Nuclear Station Unit #1

Dear Mr. Grier:

During routine station operation with the TIP detectors withdrawn, explosive charges in the TIP System shear valves were replaced with new charges. The explosive operated TIP shear isolation valves are a back-up system to be used for isolation of the TIP tubes in the event it is not possible to withdraw the detector so the normal automatic isolation system may function.

Following replacement, performance of Maintenance Procedure, N1-IMP-TIP-3 found that the ohmmeter continuity test for acceptable electric contact was unsatisfactory. The normal TIP tube isolation system was operable at this time.

It was found that the actual electrical connections to the explosive charges were not as described in the Maintenance Procedure. General Electric NED Product Service was informed of the problem and given the numbers of the drawings used to wire the system. A revision to the drawing, which had originally been issued in February 1968, was located. The configuration shown in this revision would cause the charges to fire.

The wiring for firing the charges in the TIP System was changed to match that shown in the revised drawing. All of the old charges were test fired using the wiring scheme shown in the revised drawing. They all fired successfully.

A review of the Pre-Operational Test of our Liquid Poison System showed that the explosive shear valves in that system had been operationally tested prior to startup.

In addition to the wiring problems uncovered, it was found that the 2 AMP fuses required in the squib circuits were instant blow devices instead of the slow blow type specified by the vendor. All of the fuses in the monitor/control units were replaced with new devices.

An evaluation of the safety implications is being conducted.

NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | N Y N M P 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)

7–8 | 9 LICENSEE CODE 14 | 15 LICENSE NUMBER 25 | 26 LICENSE TYPE 30 | 57 CAT 58

CON'T

01 | REPORT SOURCE L (6) | 0 5 0 0 0 2 2 0 (7) | 0 2 0 4 7 8 (8) | 0 2 2 8 7 8 (9)

7–8 | 60 | 61 DOCKET NUMBER 68 | 69 EVENT DATE 74 | 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During steady state operation, performance of surveillance test N1-ISP-

03 RE04, found RE04D with a setpoint of 3.65 PSI. T.S. 3.6.2 requires the

04 drywell high pressure switches to have a setpoint of 3.50 PSI +/- 0.053

05 PSI. This condition resulted in minimal safety implications. Redun-

06 dant instrumentation was available.

07

08

7–8 | 9 | 80

09 | SYSTEM CODE I B (11) | CAUSE CODE E (12) | CAUSE SUBCODE E (13) | COMPONENT CODE I N S T R U (14) | COMP. SUBCODE S (15) | VALVE SUBCODE Z (16)

7–8 | 9–10 | 11 | 12 | 13–18 | 19 | 20

(17) LER/RO REPORT NUMBER | EVENT YEAR 7 8 | — | SEQUENTIAL REPORT NO. 0 1 0 | / | OCCURRENCE CODE 0 3 | REPORT TYPE L | — | REVISION NO. 0

21–22 | 23 | 24–26 | 27 | 28–29 | 30 | 31 | 32

ACTION TAKEN E (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS (22) 0 0 0 0 | ATTACHMENT SUBMITTED N (23) | NPRD-4 FORM SUB. Y (24) | PRIME COMP. SUPPLIER N (25) | COMPONENT MANUFACTURER B 0 8 0 (26)

33 | 34 | 35 | 36 | 37–40 | 41 | 42 | 43 | 44–47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 RE04D, a Barton Model 289 pressure indicating switch drifted from a

11 setpoint of 3.55 PSI. It was reset to 3.50 PSI. Current surveillance

12 testing schedules are adequate to insure early detection of future

13 instrument drifts.

14

7–8 | 9 | 80

15 | FACILITY STATUS E (28) | % POWER 0 9 3 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY B (31) | DISCOVERY DESCRIPTION (32) Surveillance Testing

7–8 | 9 | 10–12 | 13–44 | 45 | 46–80

16 | ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

7–8 | 9 | 10 | 11–44 | 45–80

17 | PERSONNEL EXPOSURES NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39) NA

7–8 | 9–11 | 12 | 13–80

18 | PERSONNEL INJURIES NUMBER 0 0 0 (40) | DESCRIPTION (41) NA

7–8 | 9–11 | 12–80

19 | LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION NA

7–8 | 9 | 10–80

20 | PUBLICITY ISSUED N (44) | DESCRIPTION (45) NA | NRC USE ONLY

7–8 | 9 | 10–68 | 69–80

NAME OF PREPARER D.K. MacVittie PHONE: 315-343-2110 x 1558

GPO 917-926