

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO:
Mr. James G. Keppler

FROM:
Commonwealth Edison Company
Morris, Illinois
B. B. Stephenson

DATE OF DOCUMENT
3/31/77

DATE RECEIVED
4/7/77

LETTER
 ORIGINAL
 COPY

NOTORIZED
 UNCLASSIFIED

PROP
INPUT FORM

NUMBER OF COPIES RECEIVED
1 cc

DESCRIPTION

Ltr. trans the following:

PLANT NAME: Dresden Unit No. 2 (1-P)

RJL

ENCLOSURE

Licensee Event Report (RO 50-237/1977-10) on 3/20/77 concerning water being found to be dripping from a 1-inch drain line elbow on the "A" LPCI discharge header.....

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:	<i>Ziemann</i>
W/3 CYS FOR ACTION	
LIC, ASST.:	<i>Diggs</i>
W/1 CYS	
ACRS 16 CYS HOLDING/SENT	<i>As CAP B</i>

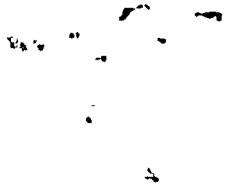
INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> REG FILE				
<input type="checkbox"/> NRC PDR				
<input type="checkbox"/> I & E (2)				
<input type="checkbox"/> MIPC				
<input type="checkbox"/> SCHROEDER/IPPOLITO				
<input type="checkbox"/> HOUSTON				
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<input type="checkbox"/> TEDESCO/MACCARY				
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<input type="checkbox"/> BAER				
<input type="checkbox"/> SHAO				
<input type="checkbox"/> VOLLMER/BUNCH				
<input type="checkbox"/> KREGER/J. COLLINS				

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: <i>Morris, Ill</i>				770970204
TIC:				
NSIC:				





Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. #77-266

March 31, 1977



Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-237/1977-10. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

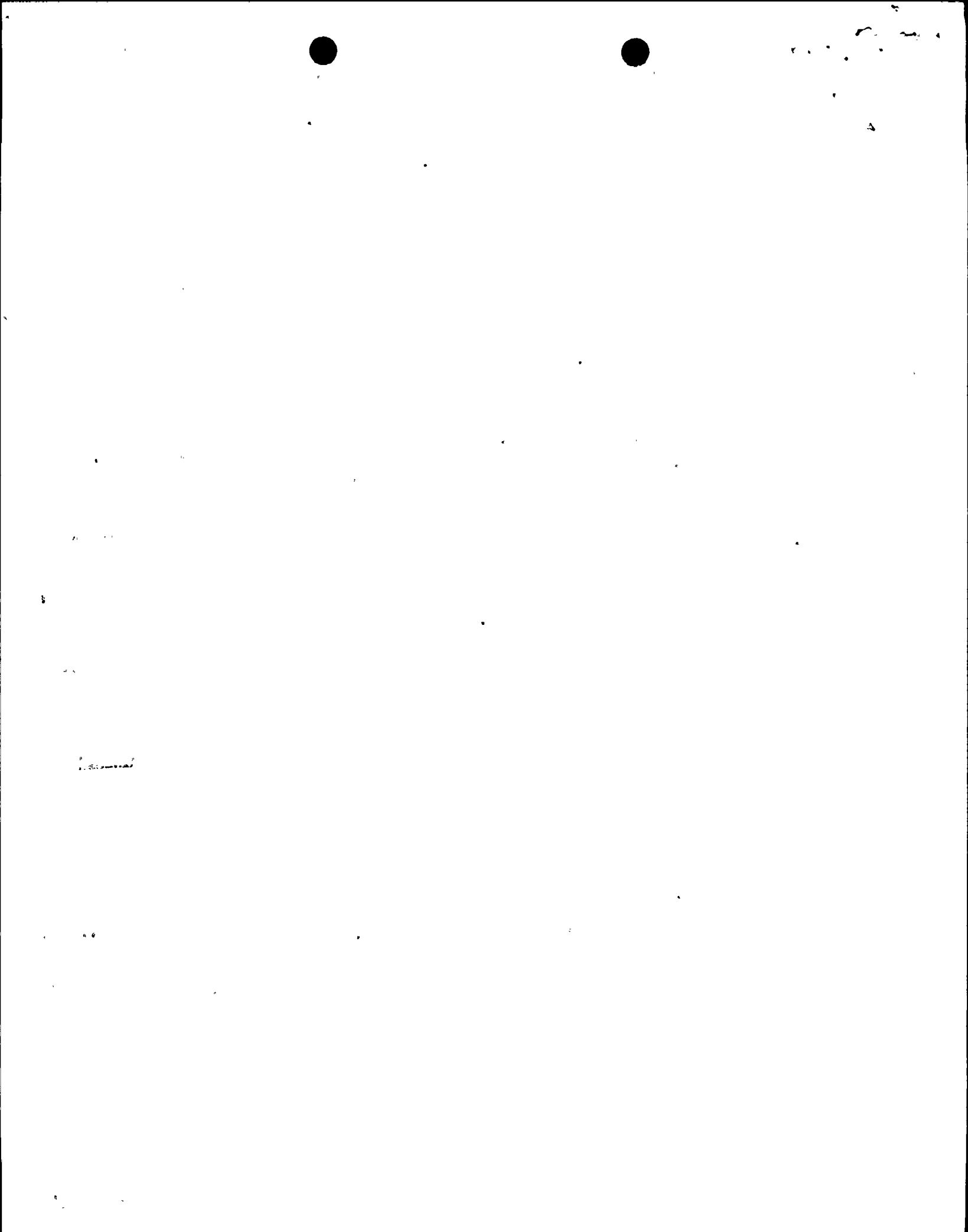
B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

770970209



INCIDENT-EVENT REPORT

CONTROL BLOCK: 1 6

PLEASE PRINT ALL REQUIRED INFORMATION

LICENSEE NAME: 01 | I | L | D | R | S | 2 | 7 8 9 14

LICENSE NUMBER: 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 15 25

LICENSE TYPE: 4 | 1 | 1 | 1 | 1 | 26 30

EVENT TYPE: 0 | 1 | 31 32

01 CONT CATEGORY: 57 58

REPORT TYPE: T 59

REPORT SOURCE: L 60

DOCKET NUMBER: 0 | 5 | 0 | - | 0 | 2 | 3 | 7 | 61 68

EVENT DATE: 0 | 3 | 2 | 0 | 7 | 7 | 69 74

REPORT DATE: 0 | 3 | 3 | 1 | 7 | 7 | 75 80

EVENT DESCRIPTION

02 | During a drywell inspection, water was found to be dripping from a 1-inch drain
7 8 9 80

03 | line elbow on the "A" LPCI discharge header (the drain line is located upstream
7 8 9 80

04 | of LPCI testable check valve A0 2-1501-25A in the drywell). Redundant low-
7 8 9 80

05 | pressure core cooling systems were available and operable. Furthermore, the
7 8 9 80

06 | safety significance of a 1-inch primary system line leak is minimal, since the
7 8 9 80

(Continued)

SYSTEM CODE: C | E | 7 8 9 10

CAUSE CODE: E | 11

COMPONENT CODE: P | I | P | E | X | X | 12 17

PRIME COMPONENT SUPPLIER: N | 43

COMPONENT MANUFACTURER: Z | 9 | 9 | 9 | 44 47

VIOLATION: N | 48

CAUSE DESCRIPTION

08 | A pinhole leak was discovered in the first 90° elbow socket weld on the 1-inch
7 8 9 80

09 | drain line. Subsequent grinding in the leakage area revealed a subsurface 1/2-inch
7 8 9 80

10 | long circumferential crack in the weld. The crack was apparently the result of
7 8 9 80

(Continued)

FACILITY STATUS: G | 7 8 9

% POWER: 0 | 0 | 0 | 10 12 13

OTHER STATUS: NA | 13 14

METHOD OF DISCOVERY: B | 44 45

DISCOVERY DESCRIPTION: NA | 46 47

FORM OF ACTIVITY RELEASED: Z | 7 8 9

CONTENT OF RELEASE: Z | 10 11

AMOUNT OF ACTIVITY: NA | 11 12 13

LOCATION OF RELEASE: NA | 44 45

PERSONNEL EXPOSURES

13 | NUMBER: 0 | 0 | 0 | 7 8 9 11

TYPE: Z | 12

DESCRIPTION: NA | 13 80

PERSONNEL INJURIES

14 | NUMBER: 0 | 0 | 0 | 7 8 9 11

DESCRIPTION: NA | 12 80

OFFSITE CONSEQUENCES

15 | NA | 7 8 9 80

LOSS OR DAMAGE TO FACILITY

16 | TYPE: Z | 7 8 9 10

DESCRIPTION: NA | 10 80

PUBLICITY

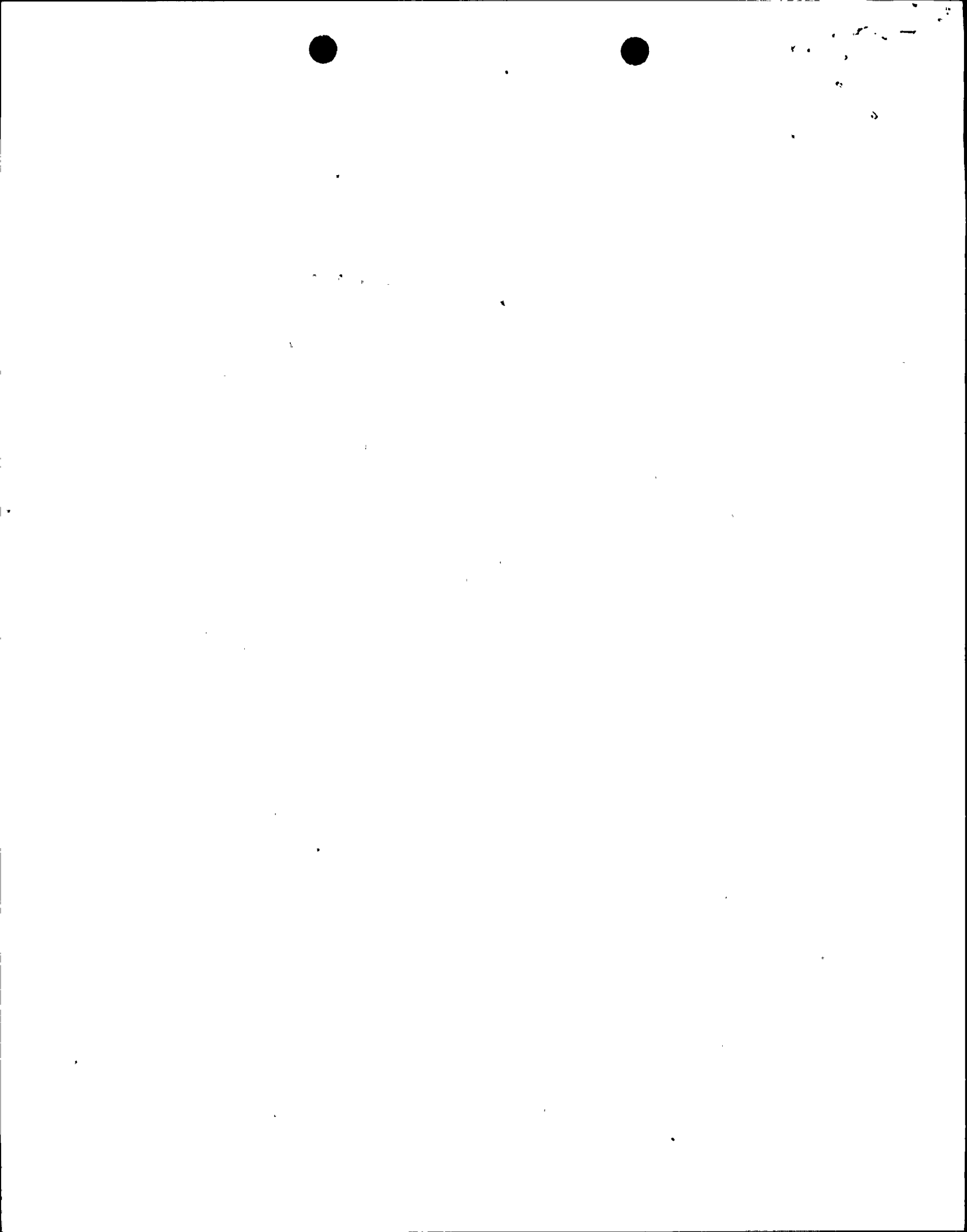
17 | NA | 7 8 9 80

ADDITIONAL FACTORS

18 | NA | 7 8 9 80

19 | 7 8 9 80

NAME: R. C. Herbert PHONE: Ext. 265



EVENT DESCRIPTION (Continued)

normal reactor feedwater system alone is capable of compensating for any leakage resulting from such a failure. This is not a repetitive occurrence. (50-237/1977-10)

CAUSE DESCRIPTION (Continued)

slag inclusion in the original weld. The weld defect was ground out, repaired, and dye-penetrant tested in accordance with approved procedures. Hydrostatic testing of the repaired weld was satisfactory.

The corresponding 1-inch drain line on the "B" LPCI discharge header was visually inspected for leaks, and both drain lines were verified to be properly supported. No further action was indicated.

RECEIVED DOCUMENT
PROCESSING UNIT

1911 APR 7 PM 1 41



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. LAXHAM

BBS Ltr. #77-266

March 31, 1977

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Enclosed please find Reportable Occurrence report number 50-237/1977-10.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

APR 6 1977



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LICENSEE EVENT REPORT

CONTROL BLOCK:

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME														LICENSE NUMBER											LICENSE TYPE					EVENT TYPE	
01	1	L	D	R	S	2	0	0	-	0	0	0	0	0	0	0	0	4	1	1	1	1	0	1							
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32						
01 CONT		CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER					EVENT DATE				REPORT DATE																
01	7	8	57	58	T	L	0	5	0	-	0	2	3	7	0	3	2	0	7	7	0	3	3	1	7	7					
7	8	9	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80					

EVENT DESCRIPTION

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7 8 9 | 80

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05 | pressure core cooling systems were available and operable. Furthermore, the
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06 | safety significance of a 1-inch primary system line leak is minimal, since the
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(Continued)

SYSTEM CODE		CAUSE CODE		COMPONENT CODE				PRIME COMPONENT SUPPLIER		COMPONENT MANUFACTURER			VIOLATION			
07	C	E	E	P	I	P	E	X	X	N	Z	9	9	9	N	
7	8	9	10	11	12	13	14	15	16	17	43	44	45	46	47	48

CAUSE DESCRIPTION

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7 8 9 | 80

(Continued)

FACILITY STATUS		% POWER			OTHER STATUS			METHOD OF DISCOVERY		DISCOVERY DESCRIPTION						
11	G	0	0	0	NA	B	NA									
7	8	9	10	11	12	13	44	45	46	47	48	49	50	51	52	53
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE								
12	Z	Z	NA	NA	NA	NA	NA	NA	NA	NA	NA	NA	NA	NA	NA	NA
7	8	9	10	11	12	13	44	45	46	47	48	49	50	51	52	53

PERSONNEL EXPOSURES

NUMBER		TYPE	DESCRIPTION													
13	0	0	0	Z	NA											
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23

PERSONNEL INJURIES

NUMBER		DESCRIPTION														
14	0	0	0	NA												
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23

OFFSITE CONSEQUENCES

15 | NA
7 8 9 | 80

LOSS OR DAMAGE TO FACILITY

TYPE	DESCRIPTION															
16	Z	NA														
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23

PUBLICITY

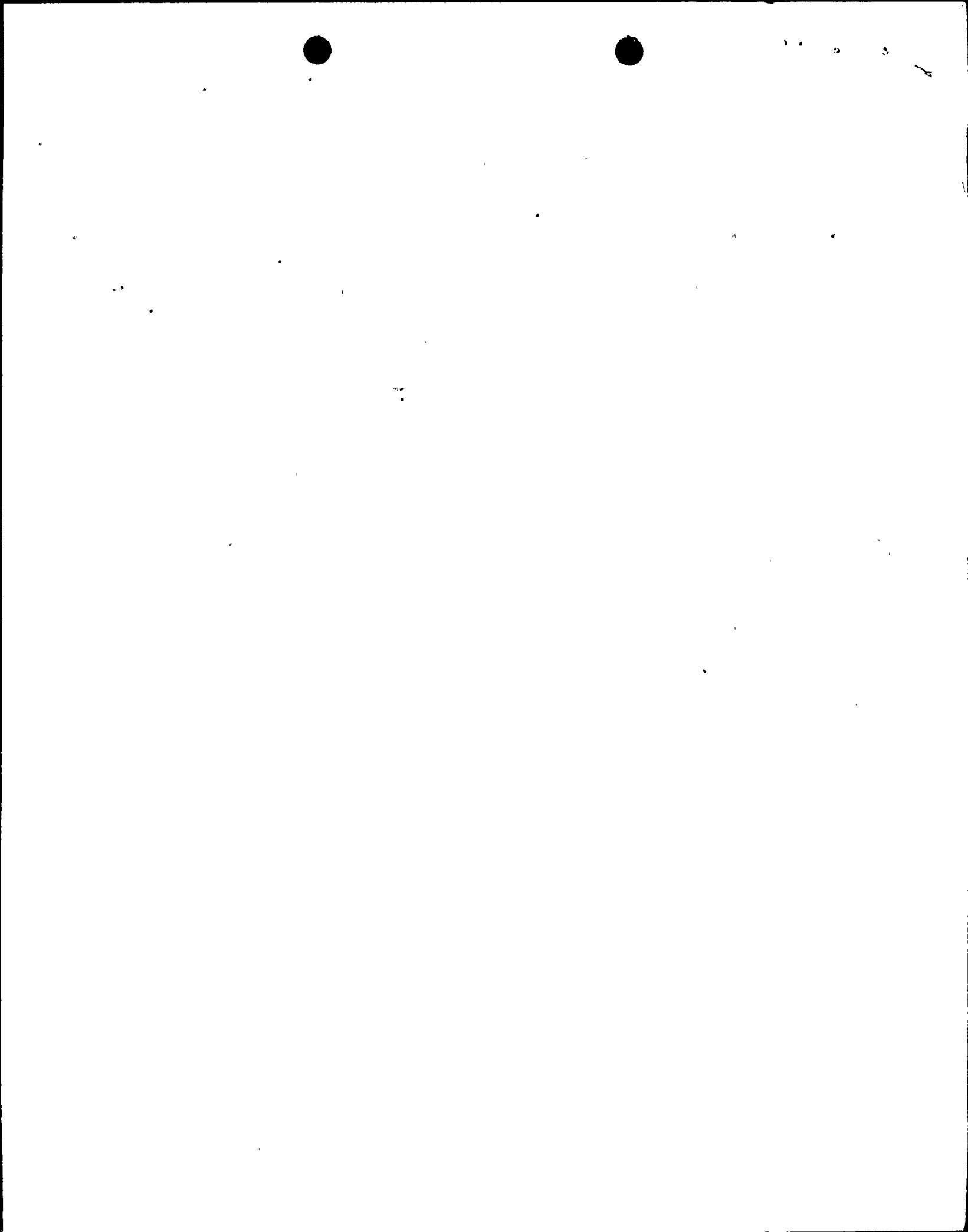
17 | NA
7 8 9 | 80

ADDITIONAL FACTORS

18 | NA
7 8 9 | 80

19 |
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NAME: R. C. Herbert PHONE: Ext. 265



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