

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO:

Mr. James G. Keppler

FROM:
Commonwealth Edison Company
Morris, Illinois
B. B. Stephenson

DATE OF DOCUMENT
4/15/77

DATE RECEIVED
4/21/77

LETTER
 ORIGINAL
 COPY

NOTORIZED
 UNCLASSIFIED

PROP
INPUT FORM

NUMBER OF COPIES RECEIVED
1 cc.

DESCRIPTION

Ltr. trans the following:

PLANT NAME: Dresden Unit No. 2 (1-P)

RJL

ENCLOSURE

Licensee Event Report (RO. 50-237/1977-13) on 4/1/77 concerning routine surveillance of the drywell floor drain sump pump flow rate on Unit No. 2 indicating a leak of reactor coolant inside the primary containment....

ACKNOWLEDGED

DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:	ZEIMANN
W/3 CYS FOR ACTION	
LIC. ASST.:	DIGGS
W/3 CYS	XXXXX
ACRS 16 CYS HOLDING/SENT	AS CAT. B

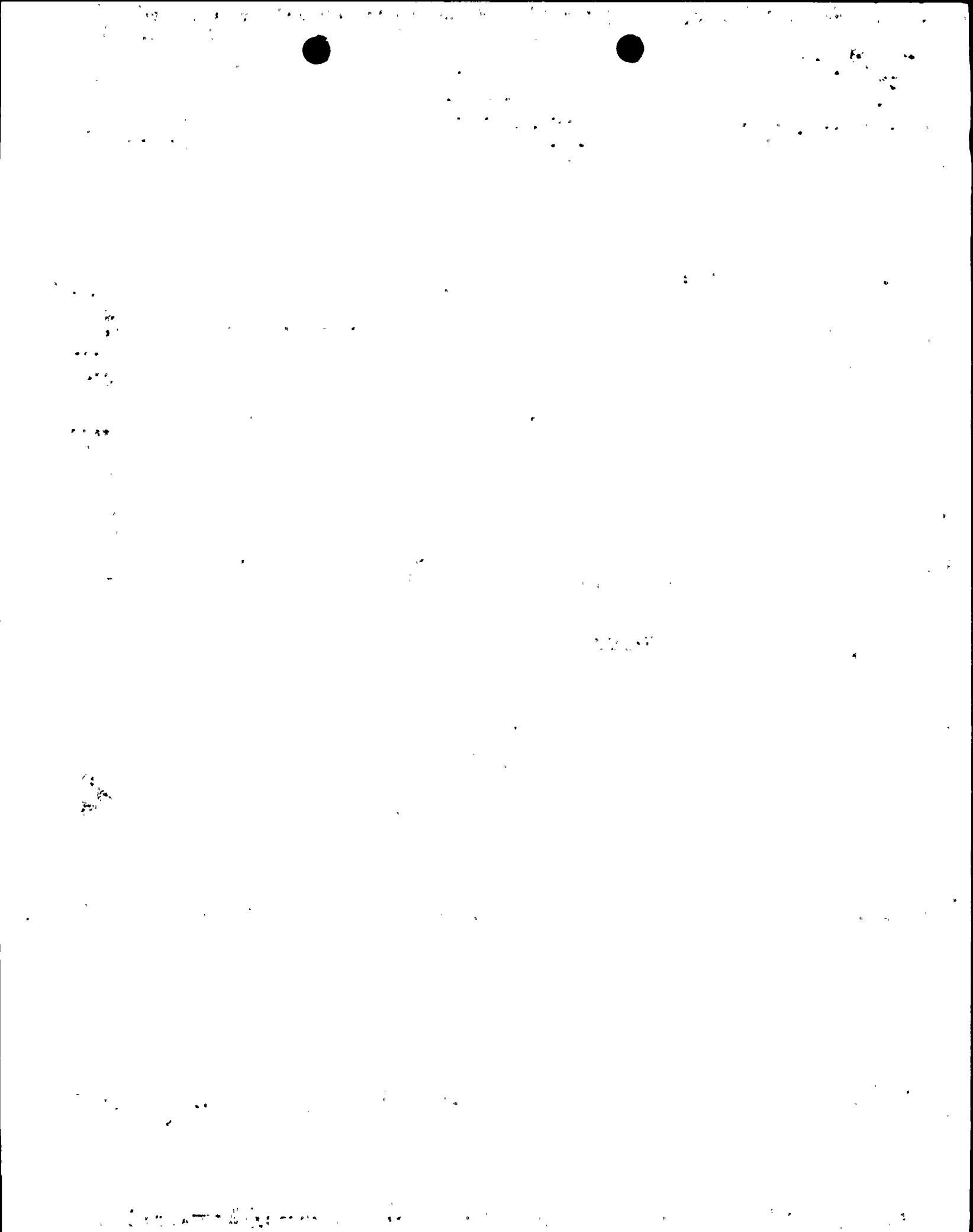
INTERNAL DISTRIBUTION

<input checked="" type="checkbox"/> REG FILE				
NRC PDR				
I & E (2)				
MIPC				
SCHROEDER/IPPOLITO				
HOUSTON				
NOVAK/CHECK				
GRIMES				
CASE				
BUTLER				
HANAUER				
TEDESCO/MACCARY				
EISENHUT				
BAER				
SHAO				
VOLLMER/BUNCH				
KREGER/J. COLLINS				

EXTERNAL DISTRIBUTION

LPDR: MORRIS ILL.			
TIC:			
NSIC:			

CONTROL NUMBER
7711570138





Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

BBS Ltr. #77-322

April 15, 1977

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



Enclosed please find Reportable Occurrence report number 50-237/1977-13.
This report is being submitted to your office in accordance with the Dresden
Nuclear Power Station Technical Specifications, Section 6.6.B.

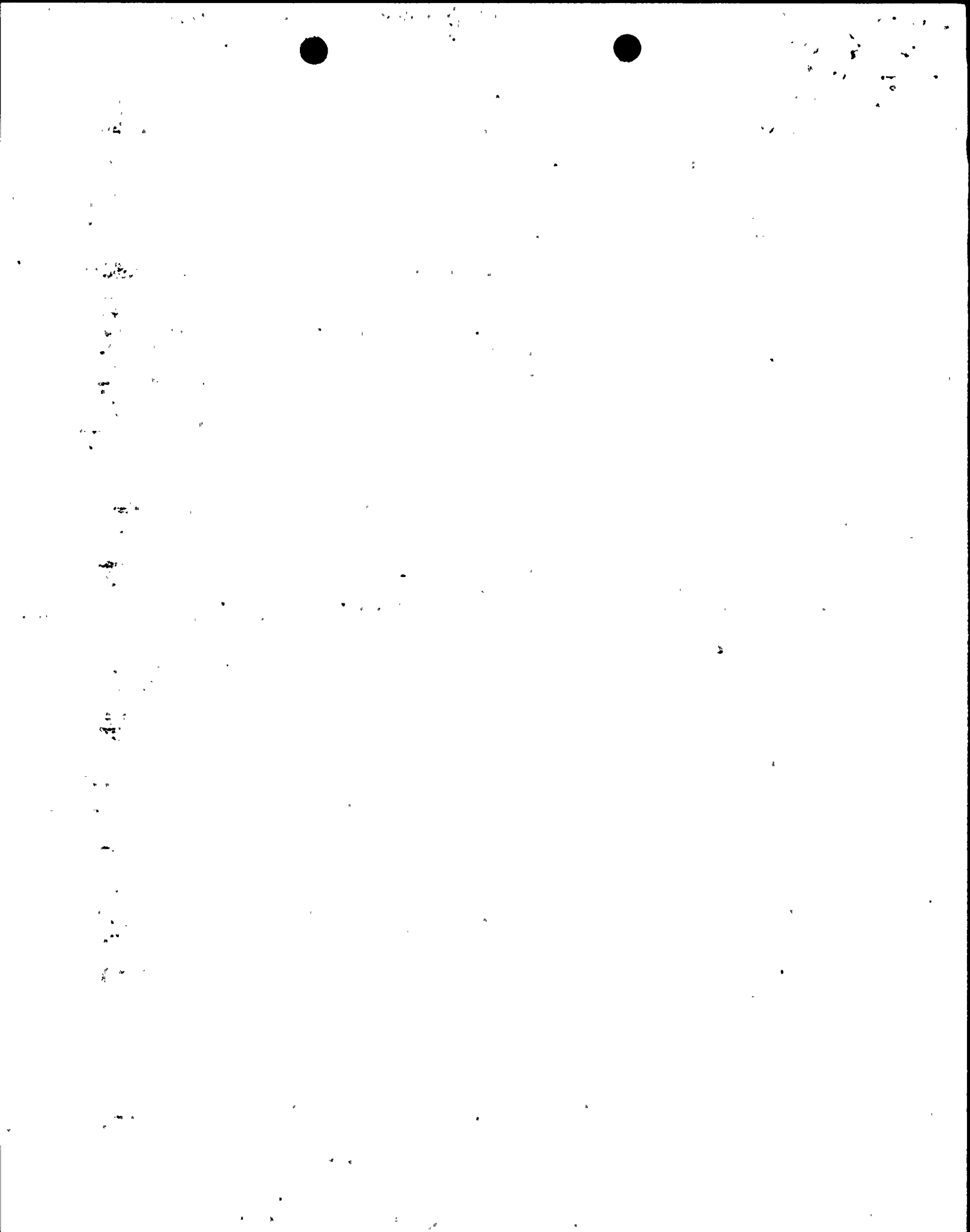
E. B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:jo

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

771150138





Commonwealth Edison

DEVIATION REPORT

DVR NO.	STA	UNIT	YEAR	NO.
D-12	-	2	-	77 - 29

PART 1 TITLE OF DEVIATION

OCCURRED	DATE	TIME
4/1/77		

Leak in 4" Bypass Loop B Water Chemistry Test Flange

SYSTEM AFFECTED

PLANT CONDITIONS

TESTING

Reactor

MODE S/U

PWR (MWT) 0

LOAD (MWE) 0

YES NO

DESCRIPTION OF EVENT

Excessive drywell leakage of approximately 2200 gal./8 hrs. caused unit shutdown.

Tech Spec limit is 2400 gal./8 hrs.

DESCRIPTION OF CAUSE

4" bypass loop B water chemistry test flange conductivity cell electrical seal blew out.

OTHER APPLICABLE INFORMATION

This was determined during drywell leak inspection.

EQUIPMENT FAILURE	<input checked="" type="checkbox"/> YES	<input type="checkbox"/> NO
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DR NO. NA

WR NO. 3074

J. S. Kolanowski
RESPONSIBLE SUPERVISOR

4/1/77
DATE

PART 2 OPERATING ENGINEERS COMMENTS

The leak was located during drywell leak inspection. Leakage was minimal and within the capability of the floor drain pump however, Tech Spec limit was being approached. The test flange will be removed and original blank flange will be reinstalled.

TYPE OF DEVIATION REPORTABLE OCCURRENCE

EVENT OF POTENTIAL PUBLIC INTEREST

TECH SPEC VIOLATION

NON-REPORTABLE OCCURRENCE

ANNUAL REPORTING

SAFETY-RELATED WR ISSUED

14 DAY	<input checked="" type="checkbox"/>
30 DAY	<input type="checkbox"/>

YES

NO

REPORTABLE OCCURRENCE NUMBER

50-237 - 77 - 13

ACTION ITEM NO.

NA

PROMPT ON-SITE NOTIFICATION

A. Roberts - Asst. Supt.

4/1/77

1500

TITLE

DATE

TIME

NA

TITLE

DATE

TIME

24-HOUR NRC NOTIFICATION

TPH

W. Shafer

4/1/77

16:10

REGION III

DATE

TIME

TGM

NA

REGION III & DOL

DATE

TIME

PROMPT OFF-SITE NOTIFICATION

F. Palmer

4/4/77

10:02

TITLE

DATE

TIME

J. Gilliom

4/4/77

10:02

TITLE

DATE

TIME

REVIEW AND COMPLETED

J. S. Kolanowski
S. Kolanowski
OPERATING ENGINEER

4/1/77

DATE

ACCEPTANCE BY STATION REVIEW AS REQUIRED

DATE

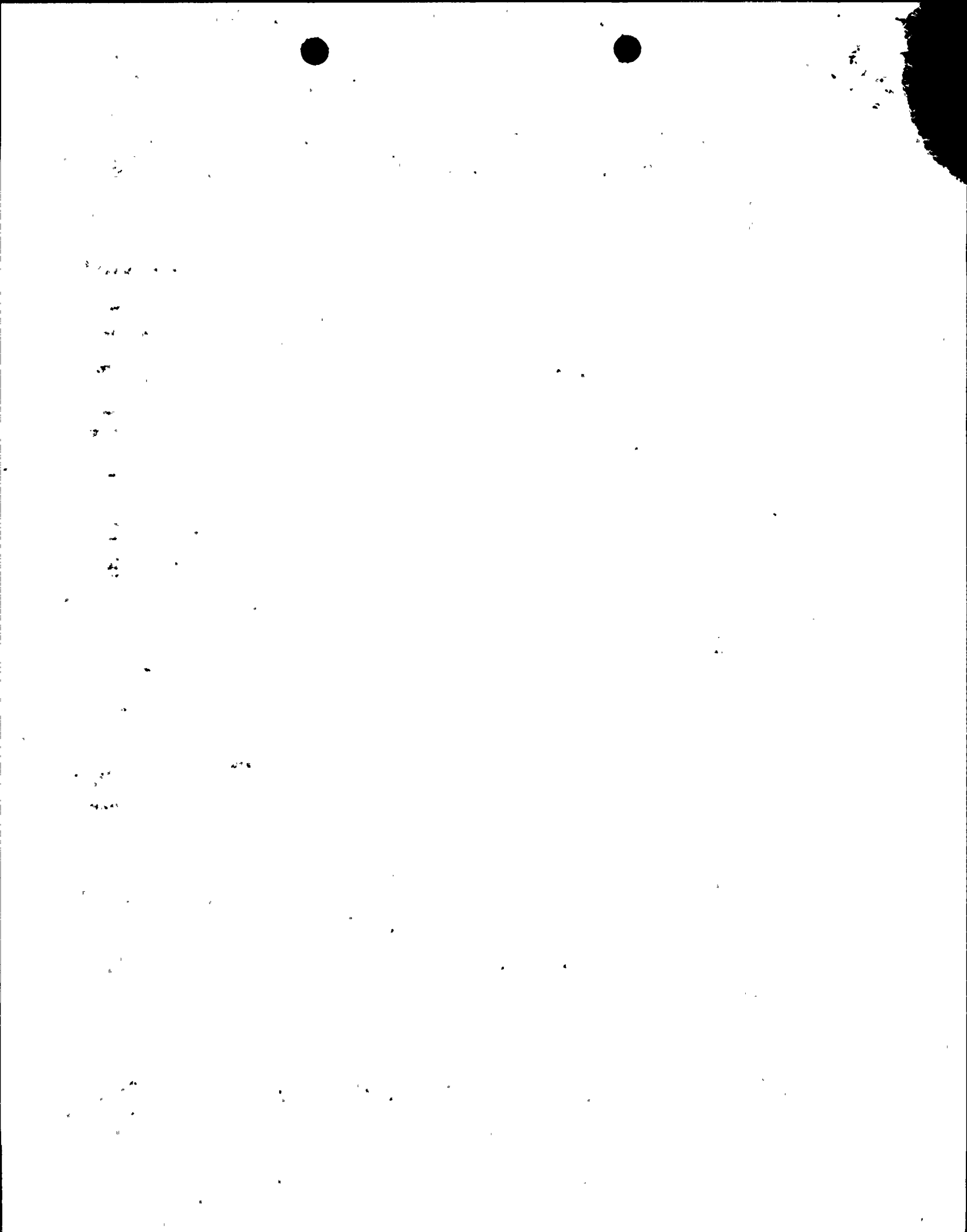
RESOLUTION APPROVED AND

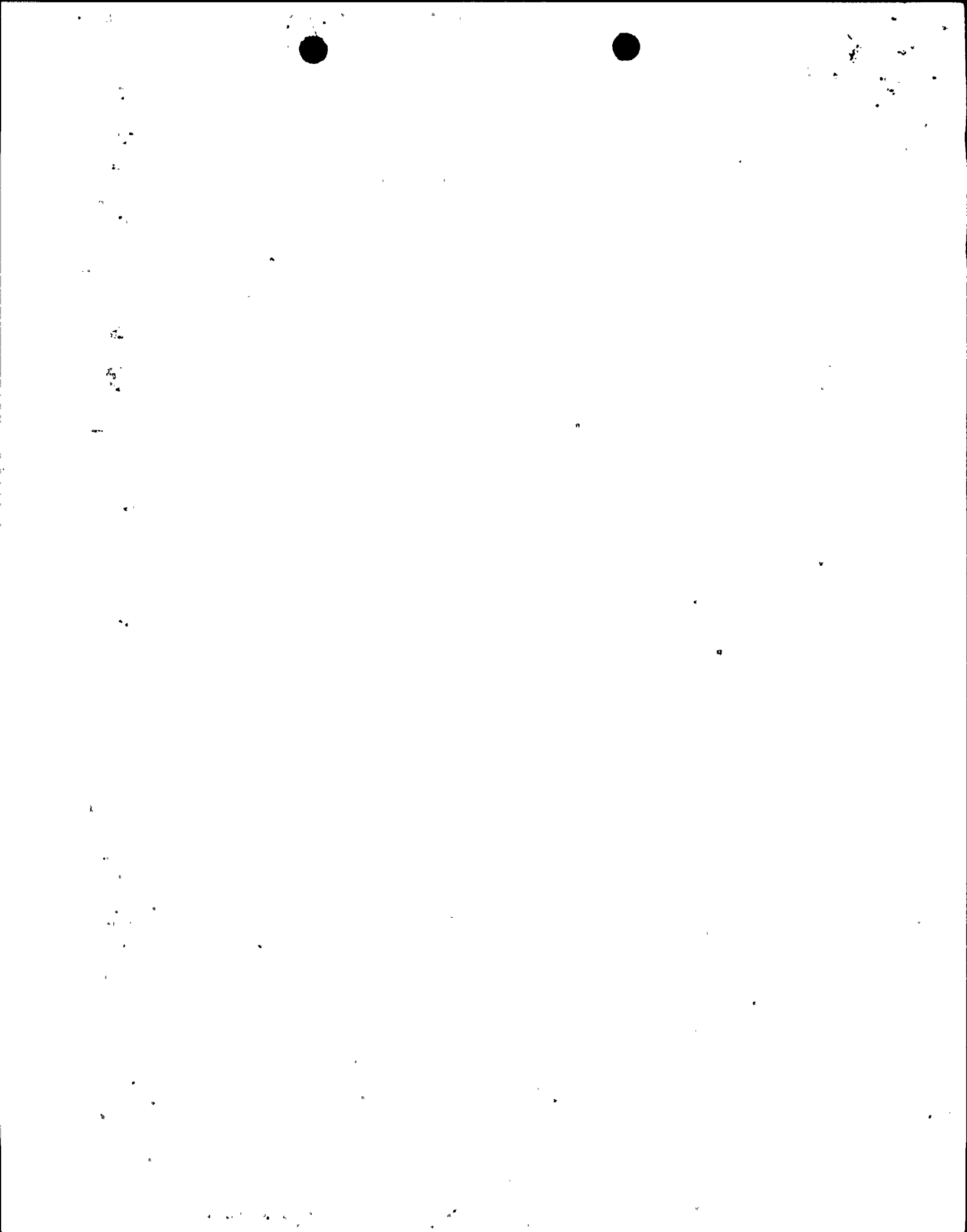
C. E. Sargent *J. S. Kolanowski*

4/15/77

4/15/77

4/15/77





EVENT DESCRIPTION (Continued)

instrumentation pipe had failed. The pipe penetrated a water chemistry test flange located in the 4-inch bypass piping around reactor recirculation pump discharge valve MO 2-0202-58. The test flange had been installed to permit chemically analyzing the reactor water in the bypass piping during operation. This testing program was completed at the end of the last fuel cycle. This event is not repetitive. (50-237/1977-13)

CAUSE DESCRIPTION (Continued)

requested that the test flange which failed, be sent to them for further investigation. In view of the fact that test flanges of the type that failed are not presently installed, no further corrective action is considered necessary.

