

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

TO:
Mr. James G. Keppler

FROM:
Commonwealth Edison Company
Morris, Illinois
B. B. Stephenson

DATE OF DOCUMENT
5/9/77

DATE RECEIVED
5/23/77

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DESCRIPTION

DO NOT REMOVE

ACKNOWLEDGED
(1-P)

PLANT NAME:
Dresden Unit No. 2

RJL

ENCLOSURE

Licensee Event Report (RO 50-237/1977-19) on 4/25/77 concerning the APRM/RBM flow-bias indication exceeding 100% on both channels A & B.....

(2-P)

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF:	ZEIMANN
W/3 CYS FOR ACTION	
LIC. ASST.:	DIGGS
W/1 CYS	
ACRS	16 CYS HOLDING/SENT AS CAT B

INTERNAL DISTRIBUTION

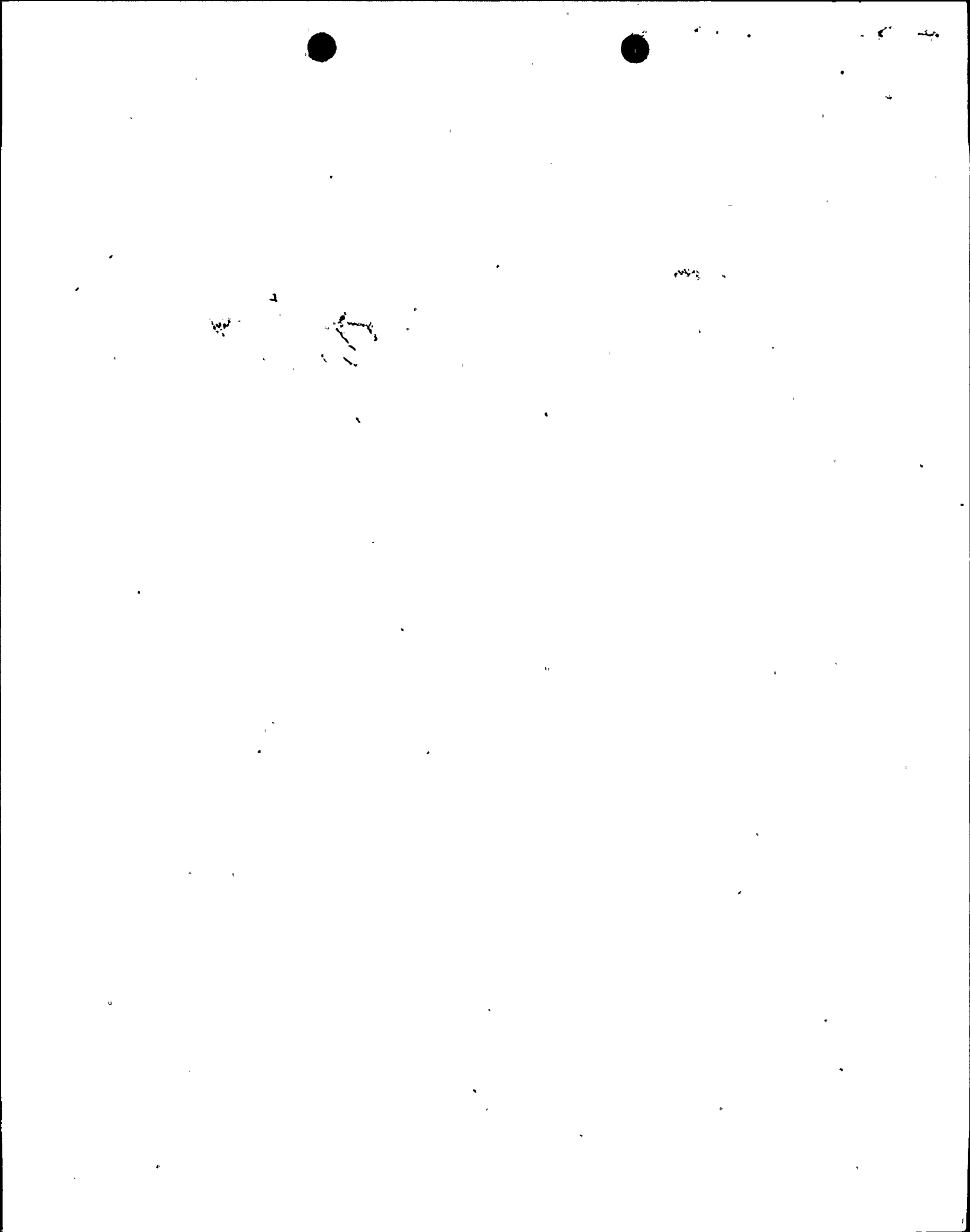
<input checked="" type="checkbox"/> REG FILE			
<input type="checkbox"/> NRC-PDR			
<input type="checkbox"/> I. & E (2)			
<input type="checkbox"/> MIFG			
SCHROEDER/IPPOLITO			
HOUSTON			
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GRIMES			
BUTLER			
HANAUER			
TEDESCO/MACCARY			
EISENHUT			
BAER			
SHAO			
VOLLMER/BUNGH			
KREGER/J. COLLINS			

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: MORRIS ILL.			
TIC:			
NSIC:			

771430052





Commonwealth Edison
 Dresden Nuclear Power Station
 R.R. #1
 Morris, Illinois 60450
 Telephone 815/942-2920

D. Latham

BBS Ltr. # 77-419

May 9, 1977



Mr. James G. Keppler, Regional Director
 Directorate of Regulatory Operations - Region III
 U. S. Nuclear Regulatory Commission
 799 Roosevelt Road
 Glen Ellyn, IL 60137

REGULATORY DOCKET FILE COPY

Enclosed please find Reportable Occurrence report number 50-237/1977-19.
 This report is being submitted to your office in accordance with the Dresden
 Nuclear Power Station Technical Specifications, Section 6.6.B.

B. B. Stephenson
 B. B. Stephenson
 Station Superintendent
 Dresden Nuclear Power Station

BBS:sm

Enclosure

cc: Director of Inspection & Enforcement
 Director of Management Information & Program Control
 File/NRC

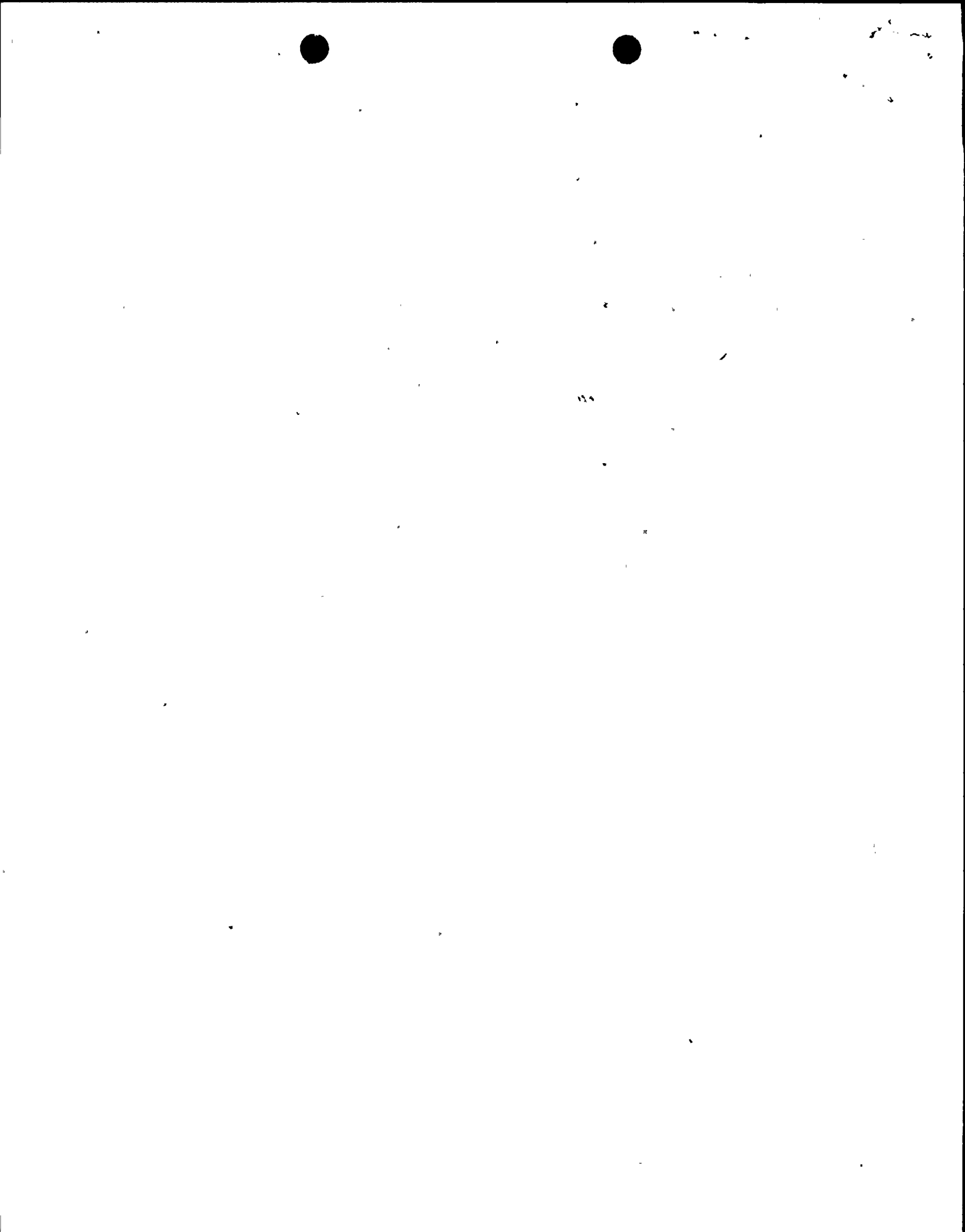
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MAY 11 1977

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EVENT DESCRIPTION (continued)

(50-237/1977-2) as a result of calibration of the total core flow indication. The safety implications of this event were minimal because safety and transient analyses do not assume a flow-biased scram function (50-237/1977-19).

CAUSE DESCRIPTION (continued)

not take into account the present correlation between recirculation drive flow and total core flow (to which the APRM/RBM flow converter readings are referenced). As a result the APRM/RBM scram trip levels could have been non-conservative at core flow rates of less than 100%.

On April 25, 1977, the APRM/RBM flow converters were readjusted to read 100% at 100% total core flow. To prevent a reoccurrence of this event, the appropriate instrument calibration procedure will be revised to include the present correlation between recirculation drive flow and total core flow.

APR 25 1977 10 3 51

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1977 MAY 24 AM 8 21

ENCLOSURE

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