

NRC DISTRIBUTION FOR PART 50 DOCKET MATERIAL

FILE NUMBER
INCIDENT REPORT

TO:
Mr. James G. Keppler

FROM:
Commonwealth Edison
Morris, Illinois
B. B. Stephenson

DATE OF DOCUMENT
8/1/77

DATE RECEIVED
8/24/77

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DESCRIPTION

PLANT NAME: Dresden Unit No. 2 (1-P)
RJL 8/25/77

ENCLOSURE

Licensee Event Report (RO 50-237/1977-21) on 6/3/77 concerning a containment cooling service water LPCI heat exchanger valve 2-1501-3A failing to open when "A" CCSW pump was started.....

(2-P)

ACKNOWLEDGED
DO NOT REMOVE

NOTE: IF PERSONNEL EXPOSURE IS INVOLVED SEND DIRECTLY TO KREGER/J. COLLINS

FOR ACTION/INFORMATION

BRANCH CHIEF: (4) DAVIS			
W/S CYS FOR ACTION			
LIC ASST.			

INTERNAL DISTRIBUTION

REG FILE			
NRC PDR			
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ROSA			

EXTERNAL DISTRIBUTION

CONTROL NUMBER

LPDR: MORRIS, III			
TIC:			
NSIC:			
ACRS (16) SENT AS CAT. B			

772370121

1954

1954

1954



1954



Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

D. Larson

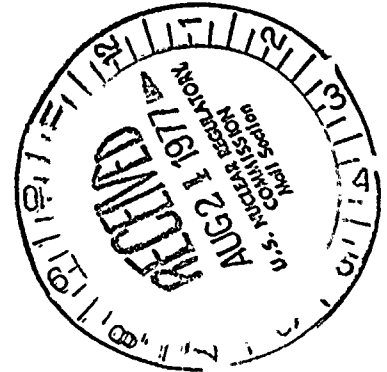
August 1, 1977

Regulatory

File-Cy

BBS LTR #77-695

Mr. James G. Keppler, Regional Director
Directorate of Regulatory Operations - Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137



Enclosed please find an update report on Reportable Occurrence report number 50-237/1977-21. This report is being submitted to your office in accordance with the Dresden Nuclear Power Station Technical Specifications, Section 6.6.B.

Arthur M Roberts
for B.B. Stephenson
Station Superintendent
Dresden Nuclear Power Station

BBS:dlz

Enclosure

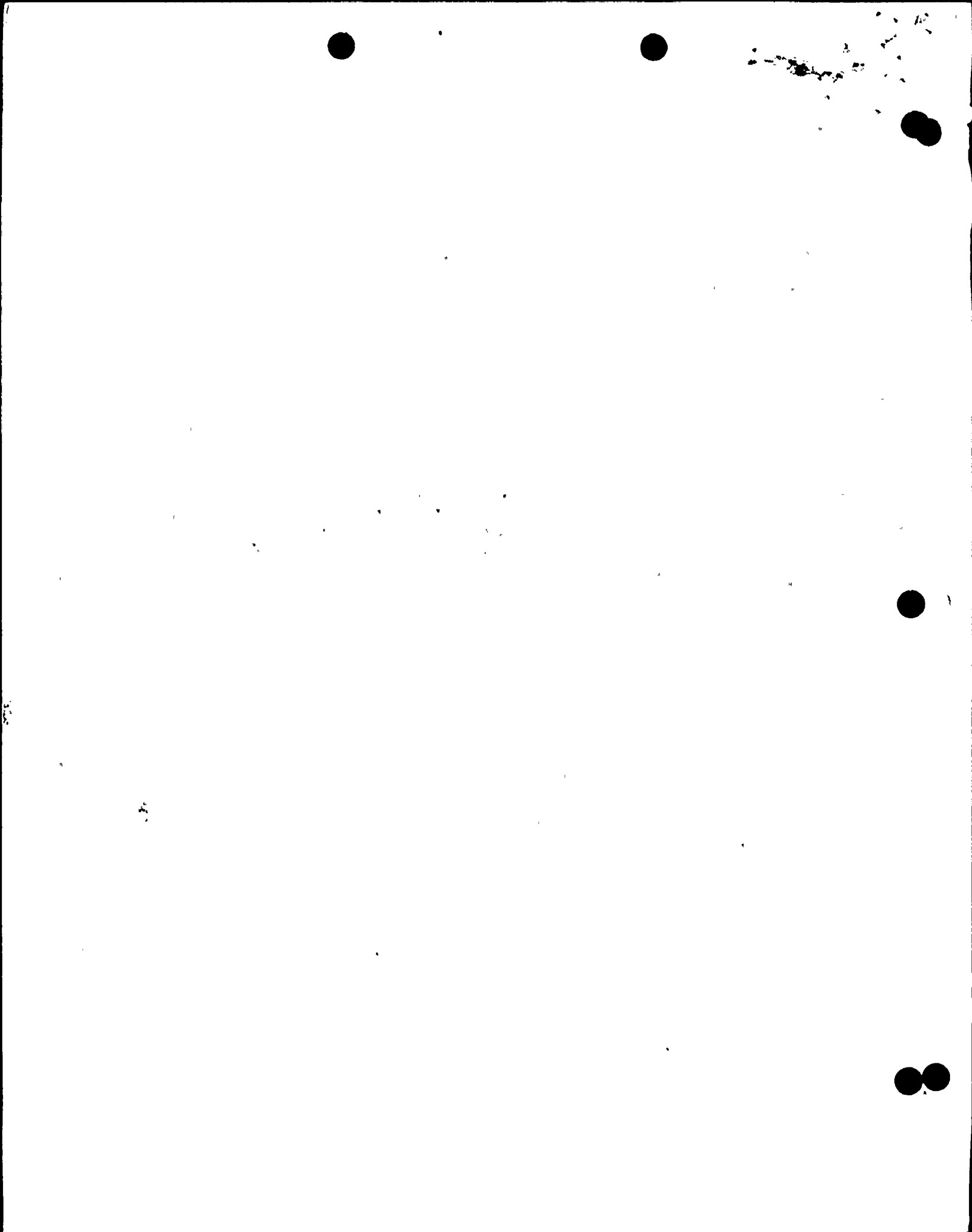
cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
File/NRC

AUG 3 1977

772370121



1952



EVENT DISCRIPTION (CONTINUED)

valve last occurred in 1975. (50-237/1977-21)

CAUSE DESCRIPTION (Continued)

position. While manually closing the valve, the stem was bent. While performing maintenance on the valve stem, the feedback potentiometer (which is mechanically linked to the valve stem) inside the limitorque valve operator became misadjusted. Thus, the valve could not respond properly to demand signals from the remote valve position controller. Adjustment of the feedback potentiometer allowed the valve to function properly. The feedback potentiometer is a Model 5611 R1K(1)L.5 Heli-pot manufactured by Beckman Instruments, Inc.. The valves are 12"-300 PSI USA standard valve assemblies manufactured by Copes-Vulcan, a division of Blaw-Knox Company.

A request has been made to the Station Nuclear Engineering Department to review this event. Our current thinking is to increase the size of the stem on the 1501-3 valves and/or to install an appropriate isolation valve in the system for maintenance. We anticipate, however, that the final resolution of this problem might take approximately 18 months due to equipment delivery times. In the interim Caution tags will be attached to the 1501-3 valves to ensure appropriate care is taken when manually shutting the valves to isolate the system.

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