

F 04/05/78

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SUBJECT:

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LICENSEE EVENT REPT (RO 50-220/77-32) ON 07/01/77 CONCERNING DURING
REFUELING, A WATER PHASE MATERIAL SAMPLE IN THE REACTOR VESSEL WAS NOT
INSPECTED AS REQUIRED BY TECH SPEC 4.2.2.D...W/ATT LER 77-32 AND W/OUT LERS
77-33, 77-36 & 77-37.

PLANT NAME: NINE MILE PT - UNIT 1

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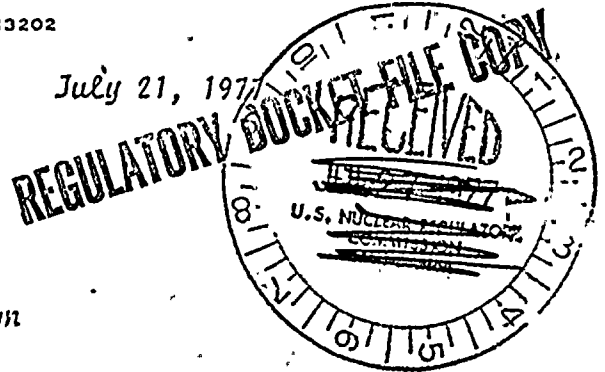
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US HRC
DISTRIBUTION SERVICES
BRANCH

July 21, 1977



Mr. James P. O'Reilly
Director
United States Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA. 19406

RE: Docket No. 50-220

Dear Mr. O'Reilly:

In accordance with the Nine Mile Point Nuclear Station Unit #1 Technical Specifications, we hereby submit Licensee Event Reports, LER's 77-32, 77-33, 77-36 and 77-37.

These reports were completed in the format designated in the Licensee Event Report Instruction Booklet 00E-SS-001, dated October 1974, revised December 8, 1975.

Very truly yours,

ORIGINAL SIGNED BY R.R. SCHNEIDER

R.R. Schneider
Vice President -
Electric Production

MAS/mtm

Enclosure (3 copies)

cc: Director, I&E (40 copies)
Director, MIPC (3 copies)

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NOTICE

1911

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LICENSEE EVENT REPORT

CONTROL BLOCK: 0118121918

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: NYNMP1; LICENSE NUMBER: 00-00000-00; LICENSE TYPE: 41111; EVENT TYPE: 03; CATEGORY: 57; REPORT TYPE: L; REPORT SOURCE: L; DOCKET NUMBER: 050-0220; EVENT DATE: 070177; REPORT DATE: 072277

EVENT DESCRIPTION (LER 77-32). DURING REFUELING, A WATER PHASE MATERIAL SAMPLE IN THE REACTOR VESSEL WAS NOT INSPECTED AS REQUIRED BY T.S. 4.2.2. D. OTHER INSPECTIONS AND ANALYSES INDICATE THAT NO DETERIORATION OF THE SENSITIZED STAINLESS STEEL HAS OCCURRED.

SYSTEM CODE: CA; CAUSE CODE: A; COMPONENT CODE: VESEEL; PRIME COMPONENT SUPPLIER: N; COMPONENT MANUFACTURER: G080; VIOLATION: Y

CAUSE DESCRIPTION INABILITY TO REMOVE AND COLLECT MATERIAL SAMPLE PACKET WITH TOOLS CURRENTLY AVAILABLE. ATTEMPTS WILL BE MADE FOR NEXT REFUELING TO DESIGN A TOOL, OR PLACE A NEW SAMPLE IN ACCESSIBLE LOCATION.

FACILITY STATUS: H; % POWER: 000; OTHER STATUS: NA; METHOD OF DISCOVERY: A; DISCOVERY DESCRIPTION: NA; FORM OF ACTIVITY RELEASED: Z; CONTENT OF RELEASE: Z; AMOUNT OF ACTIVITY: NA; LOCATION OF RELEASE: NA

PERSONNEL EXPOSURES: NUMBER: 000; TYPE: Z; DESCRIPTION: NA

PERSONNEL INJURIES: NUMBER: 000; DESCRIPTION: NA

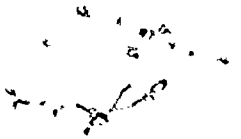
CONSEQUENCES PROBABLE: NA

LOSS OR DAMAGE TO FACILITY: TYPE: Z; DESCRIPTION: NA

PUBLICITY: NA

ADDITIONAL FACTORS: NA

NAME: E. Sobel 7/27/77; PHONE: 4927735



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EVENT DESCRIPTION

During a routine refueling outage, a water phase material sample in the reactor vessel was not inspected as is required by Technical Specification 4.2.2.d. Inspections of the other material samples in steam and steam/water phase areas, and analysis of other conditions such as primary coolant chemistry indicate that no deterioration of the sensitized stainless steel has occurred. A Technical Specification change has been recommended to eliminate the requirement for a water phase material sample inspection for the current outage.

CAUSE DESCRIPTION

Due to the location and radioactivity of the material sample packet, it was not possible to remove and collect it with any tools currently available. During the next scheduled refueling outage, attempts will be made to design a tool for such a purpose, or a new material sample will be placed in the reactor in a location which permits retrieval.

