A 04/26/78

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DOCDATE: 04/18/78
DATE RCVD: 04/26/78

DOCTYPE: LETTER NOTARIZED: NO
SUBJECT:
RESPONSE TO ORAL REQUESTS MADE BY NRC...FURNISHING INFO CONCERNING SUBJECT FACILITY, (UNIT 2) COOLING TOWER.

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PLANT NAME: NINE MILE POINT - UNIT 2

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********** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS **********

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NIAGARA MOHAWK POWER CORPORATION

NIAGARA MOHAWK

300 ERIE BOULEVARD, WEST
SYRACUSE, N. Y. 13202

April 18, 1978

Mr. Edson G. Case, Acting Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

RECEIVED DISTRIBUTION SERVICES UNIT
1978 APR 26 AM 8 46
US NRC DISTRIBUTION SERVICES BRANCH

Dear Mr. Case:

Re: Nine Mile Point Unit 2
Docket No. 50-410

The following information responds to oral requests made by your staff regarding the Nine Mile Point Unit 2 cooling tower.

Section 8.4.1.3.11 of the Environmental Report-Construction Permit Stage and Figure 14 of the "Report on Circulating Water Cooling System Employing a Natural Draft Cooling Tower," July 1976, do not agree with respect to the predicted area affected by plant noise. The data contained in the latter report are more recent and accurate. These values should be used for plant noise evaluation. The major reasons for the differences in the predictions are as follows:

1) The Ellis Method[1] used to evaluate noise for the Environmental Report was the best method available at that time. This method was developed for small cooling towers. The Ellis Method overpredicts noise for large cooling towers. The predictions in the July 1976 report[2,3] were made using a new method based upon field measurements of a number of large cooling towers.

2) The 1976 noise calculations include the effect of atmospheric adsorption for transformer noise. The Environmental Report predictions did not and are therefore louder.

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The discharge ΔT calculated from the heat addition to the lake value (0.74×10^9 Btu/hr) on Page 6 of the "Report on Circulating Water Cooling System Employing a Natural Draft Cooling Tower," July, 1976, does not agree with the value in Response 14 of our September 30, 1977 submittal. The 19F ΔT indicated in this latter report is the correct value and incorporates specific cooling tower vendor predictions. The discharge ΔT of 19F is an annual average based on a discharge flow of 25,000 gallons per minute. The earlier calculations were not based upon a specific tower since the vendor had not been selected at that time.

The worst case lake surface temperature increase is about 2.7F. This corresponds to a combined plant discharge ΔT of 30.6F and a flow of 18,020 gallons per minute. These calculations are based upon conservative worst case conditions of 32F lake temperature and a wet bulb air temperature of 57F at 45 percent relative humidity. Air temperature was selected from Rochester weather data for the period 1933 to 1972, and lake temperature was based upon 1972 weather conditions.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION

Gerald K. Rhode, Vice President
System Project Management

NLR/szd

REFERENCES

1
R. M. Ellis, Cooling Tower Noise Generation and Radiation Journal of Sand and Vibration, Volume 14(2), Pages 171-182, 1971.

2
Capano, G. A.; Bradley, W.E., "Radiation of Noise From Large Natural Draft and Mechanical Draft Cooling Towers" ASME Paper No. 74-WA-HT-55.

3
Capano, G.A.; Bradley, W.E., "Noise Predicting Techniques for Siting Large Mechanical Draft Cooling Towers" presented at the 38th Annual Meeting of American Power Conference, April, 1976.

DISTRIBUTION:
Docket files
NRC PDR
Local PDR
SVarga
LPM: Bill Kane
MService
LWR-4
Attorney OELD

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APR 2 1 1978

Docket No. 50-410

Niagara Mohawk Power Corporation
ATTN: Mr. Gerald K. Rhode, Vice President
Systems Project Management
300 Erie Boulevard West
Syracuse, New York 13202

Gentlemen:

SUBJECT: REVISED INTRUSION DETECTION SYSTEMS HANDBOOK - Nine Mile Point 2

Transmitted under separate cover is a revision of the Intrusion Detection Handbook, SAND 76-0554, which may have been sent to you in May 1977. This handbook revision was prepared by the Facilities Protection Department, Sandia Laboratories, Albuquerque, New Mexico under contract with the Division of Safeguards and Security, Department of Energy (DOE) and is made available for your information as a reference source for use in the design, installation, and operation of intrusion detection systems. The attached revision supersedes the previous version.

In addition, we are also including two other reports in the same series: "Nuclear Safeguards Technology Handbook" dated December 1977, which outlines current safeguards technology development programs, and "Entry Control System Handbook" dated September 1977, which describes systems used to control access to and egress from controlled areas.

These documents are marked "Official Use Only." In releasing them to you, we request that distribution of the documents be controlled so that subsequent revisions may be sent to all recipients. Feedback from recipients of these handbooks should be addressed to Dr. Samuel C. T. McDowell, Assistant Director for Research and Development, Office of Safeguards and Security, Department of Energy, Washington, D.C. 20545.

Sincerely,

D. B. Vassallo, Assistant Director
for Light Water Reactors
Division of Project Management

Enclosures: (3)
Under separate cover

cc w/o enclosures:
See next page

Niagara Mohawk Power Corporation

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T. K. DeBoer, Director
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Swan Street Building
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Empire State Plaza
Albany, New York 12223

MEETING SUMMARY - Mark II

Docket File
NRC PDR
Local PDR
TIC
NRR Reading
LWR-#1 File
E. Case
R. Boyd
R. DeYoung
D. Vassallo
J. Stolz
K. Kniel
O. Parr
S. Varga
L. Crocker
D. Crutchfield
F. Williams
R. Mattson
H. Denton
D. Fuller
Project Manager: I. Peltier
Attorney, ELD
E. Hylton
IE (3)
ACRS (16)
L. Dreher
NRC Participants:
S. Rubenstein

4/18/78

ma4

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APR 18 1978

Docket Nos: 50-358
50-387
50-352/353
50-367
50-373/374
50-388
50-410
50-322

APPLICANTS: Members of Mark II Owner's Group

SUBJECT: MEETING WITH MARK II OWNER'S GROUP AND GENERAL ELECTRIC COMPANY TO DISCUSS THE MARK II CONTAINMENT POOL DYNAMICS LOAD PROGRAM - APRIL 6, 1978 - SAN JOSE

Background:

The staff's Mark II Containment Task Group (Task Action Plan No. A-8) met in Livermore, California and San Jose, California on April 5th and April 6th respectively to discuss the Livermore Fluid Structure Interaction Program and Submerged Structure Drag Loads Program and the Owner's Group Monticello SRV, Fluid Structure Interaction Studies, the staff's Pool Temperature Limits review, the 4T Fluid Structures Interaction Studies and other matters related to Mark II Containment Pool Dynamics. These meetings will be reported later when information becomes available from the Task Group. The DPM Mark II coordinator attended the San Jose meeting only.

Summary:

General Electric made a presentation on the effort underway to show that 4T impulse loads at the wall result in responses that will be bounded by the preveously defined bounding loads. GE will informally write up what it is doing in this effort and provide the information to the staff. Another meeting will be held in Bethesda in about three weeks to present the results.

GE made a presentation on the FSI - SRV work using the Montecello bench work study to show that flexibility in the pool wall does not introduce lack of conservatism in comparison with the Livermore work.

The staff reported its preliminary conclusion that data are not adequate to support the pool temperature limit of 150°F for the ramshead. A position paper on this matter is being reviewed by management within the NRC.

The discussion on impact loads was deferred until the end of April at which time GE will address this problem.

All Mark II owners who originally planned to use the ramshead device have not made the decision to switch to quencher devices. The Cincinnati Gas and Electric Company (Zimmer) has decided to switch. It intends to continue plant analyses and modification on the basis of its own definition of ramshead bounding loads in anticipation that the quencher will provide additional margin. The staff cautioned CG&E that the DFFR loads are not necessarily acceptable to the staff and that the burden is on the applicant to establish that quencher loads are less than whatever bounding loads are used by the applicant.

At this time the Mark II owners plan to address SRV second actuation problems on an individual plant basis.

D A Peltier

I. Peltier, Project Manager
Light Water Reactors Branch No. 1
Division of Project Management

cc: See Following Pages

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APR 18 1978

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CENTRAL FILES

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
631 PARK AVENUE
KING OF PRUSSIA, PENNSYLVANIA 19406

April 14, 1978

Docket No. 50-410

Niagara Mohawk Power Corporation
ATTN: Mr. G. K. Rhode
Vice President
System Project Management
300 Erie Boulevard, West
Syracuse, NY 13202

Gentlemen:

Enclosed is IE Bulletin No. 78-05 which requires action by you with regard to your power reactor facility(ies) having an operating license or a construction permit.

Should you have questions regarding this Bulletin or the actions required, please contact this office.

Sincerely,

Robert T. Carlson for Boyce H. Grier
Director

Enclosures:
1. IE Bulletin No. 78-05
2. List of IE Bulletins Issued in 1978

cc w/encls:
Eugene B. Thomas, Jr., Esquire

aoz
mail
WL

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D. C. 20555

IE Bulletin No. 78-05
Date: April 14, 1978

MALFUNCTIONING OF CIRCUIT BREAKER AUXILIARY CONTACT MECHANISM - GENERAL ELECTRIC MODEL CR105X

Description of Circumstances:

The Sacramento Municipal Utility District recently reported a problem encountered with the operation of the GE Model CR105X auxiliary contact mechanism installed in a 480 volt circuit breaker. Investigation into the cause for the inability to shutdown a booster supply fan (SF-A-7) in the control room emergency air conditioning system revealed that an auxiliary contact (GE Model CR105X) had failed in the closed position, preventing the fan's power supply circuit breaker from opening. The specific cause for failure was binding of the plunger arm due to burrs and nicks on its surface.

An investigation was conducted by the licensee to determine the extent of usage of this type auxiliary contact in other circuits throughout the reactor power plant. Approximately fifty (50) positions in the nuclear service motor control centers were identified as having a similar type auxiliary contact mechanism. It was also determined that many of the affected systems which require contact operation similar to that described above, either permit or provide a safety feature function during emergency conditions. An example of this type application is auxiliary contacts that must open to permit closing of certain safety related valves from 480 volt motor control centers.

The attached GE Service Advice Letter and associated instruction/drawing sheet were sent to all nuclear power reactor facilities by GE Field Service Offices. The letter, together with the instruction/drawing sheet identifies the problem and provides the recommended corrective action.

Action To Be Taken By Licensees and Permit Holders:

For all power reactor facilities with an operating license or construction permit:

1. If you have received the enclosed General Electric letter and instruction/drawing sheet addressing the auxiliary contact mechanism problem, and if you have these devices in use at your facility, it is requested that you describe what corrective action you have taken.

2. If you have not received the enclosed GE documents before, it is requested that you describe what action you plan to take if the GE CR105X auxiliary contact mechanism is in use or planned for use in safety systems at your facility(ies).

3. Facilities having an operating license should report in writing, within 45 days, and facilities with construction permits within 60 days, the results of action taken or planned with regard to Items 1 and 2 above. Your written reply should also include the date when such actions were or will be completed. Reports should be submitted to the Director of the appropriate NRC Regional office and a copy should be forwarded to the U. S. Nuclear Regulatory Commission, Office of Inspection and Enforcement, Division of Reactor Operations Inspection, Office of Inspection and Enforcement, Washington, D. C. 20555.

Approved by GAO, B180225 (R0072); clearance expires 7/31/80. Approval was given under a blanket clearance specifically for identified generic problems.

Attachments:
1. GE Letter
2. GE Control and Instruction Sheet

control

INSTRUCTIONS

105X AUXILIARY CONTACT
(CHANGEOUT OF PLUNGER ARM AND INSULATION)

for nema size 0 &1 magnetic starters and contactors

REMOVING OVERLOAD RELAY BRACKET (SEE FIG. 1)

1. Remove all electrical power from device.
2. Disassemble device by pressing on the coil while pulling out on the coil retainers. Move retainers back and away from coil.
3. Grasp movable portion of magnet and pull out, removing magnet assembly, coil, molded cover and movable arm in one motion. (NOTE: On reversers or multispeed starters, this magnet assembly is linked to the mechanical interlock arm between contactors and should be tipped toward interlock arm to facilitate easy removal.)
4. Remove overload relay bracket by sliding it out of the keyed slots in the molded base. (NOTE: If any wires are too short to allow easy removal, temporarily disconnect these.)
5. Remove overload relay from bracket by loosening screw on bottom of overload bracket.

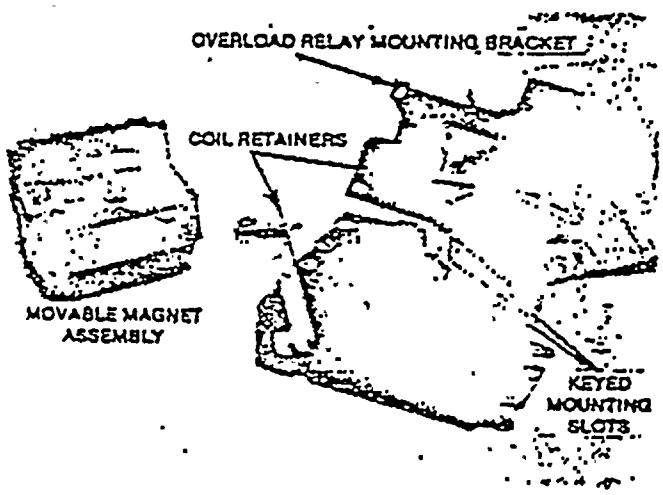

FIGURE 1
REMOVING OVERLOAD RELAY MOUNTING BRACKET

CHANGING PLUNGER ARM & INSULATION (SEE FIG. 2)

1. Slide plunger assembly out of auxiliary contact block, by lifting up and out.
2. Select the new plunger arm that is bent the same way as the one installed.
3. Loosen screw on plunger to release metal arm but DO NOT REMOVE SCREW FROM PHENOLIC PLUNGER (OR PLUNGERS).
4. Mount new plunger arm on plunger. Fasten arm to plunger with screw (or plunger post if installing double aux. blocks).
5. Apply thin coat of aeroshell grease along plunger guides.
6. Reinstall plunger assembly in contact block.
7. Reassemble overload to mounting bracket. Remove overload insulation and install new. (NOTE: Overload insulation must be removed and replaced with new insulation supplied with kit.)
8. Remount complete assembly on contactor base.
9. Reconnect any wires previously removed.
10. Reassemble magnet assemoly to contact base (on reversers or multispeeds be sure to interlock this assembly with mechanical interlock arm. Check for free movement of armature assemblies and mechanical interlocks).
11. Observe that auxiliary contact plunger assemblies are operating properly.

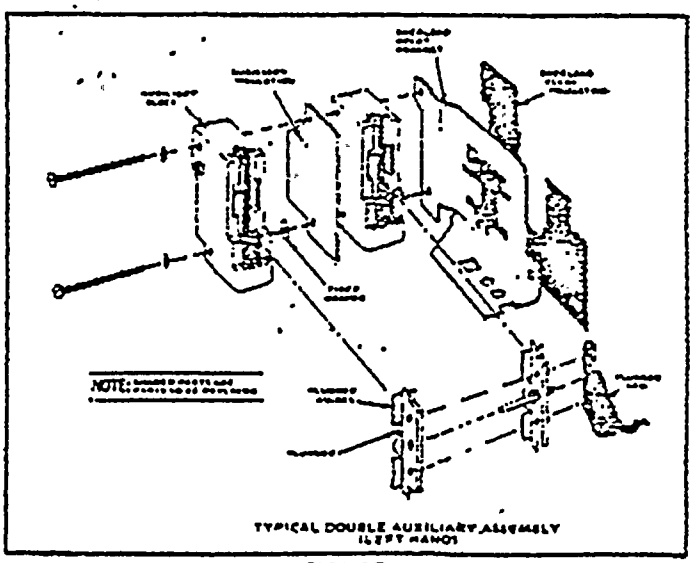

FIGURE 2

GENERAL ELECTRIC COMPANY
GENERAL PURPOSE CONTROL DEPARTMENT

GENERAL ELECTRI

It has been brought to our attention that a potential binding problem may exist with the 100 Line CR105X... auxiliary interlocks mounted on NEMA size 00, 0 and 1 contactors, starters, or reversers used in control equipment manufactured by General Electric Co. (including 7700 Line motor control centers) and other control equipment manufacturers. The interlocks involved are used on the following devices.

CR105A, -B, -C, -H, -J, -K, -R, -S with additional suffix letters and numbers (contactors)

CR106A, -B, -C, -H, -J, -K, -R, -S with additional suffix letters and numbers (starters)

CR109A, -B, -C, -H, -J, -K, -R, -S with additional suffix letters and numbers (reversing starters)

If any of these devices are currently used in your installations, you should obtain replacement parts to correct this condition by forwarding the following information to my attention:

1. Complete catalog (CR) numbers on device nameplate.

2. Quantity of each device catalog number.

3. Are the auxiliary interlocks mounted on the left side, right side, or both sides of the device? How many are there at each location?

Since the General Purpose Control Department of the General Electric Co. does not know all applications of these interlocks, it is assumed that some may be in "critical" applications and should be changed out as soon as possible, while those in non-critical applications may be changed out at some routine maintenance period.

Since these interlocks are on devices which have been in service for over one (1) year and are out of our normal warranty period, we feel a fair and equitable solution would require that the necessary parts and instructions for the suggested changeout be provided by GE at no expense to the user. At your earliest convenience, please provide me with the information requested in paragraph 2 of this letter so that the necessary parts may be provided.

This information should be forwarded to:

General Electric Company
P. O. Box 2913
Bloomington, Illinois 61701

Attention: Mr. John E. Corbitt
Product Service

(Dial Comm: 8*325-4416, or 4294)

ENCLOSURE 2

LISTING OF IE BULLETINS
ISSUED IN 1978

Bulletin No.	Subject	Date Issued	Issued To
78-01	Flammable Contact - Arm Retainers in G.E. CR120A Relays	1/16/78	All Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)
78-02	Terminal Block Qualification	1/30/78	All Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)
78-03	Potential Explosive Gas Mixture Accumulations Associated with BWR Offgas System Operations	2/8/78	All BWR Power Reactor Facilities with an Operating License (OL)
78-04	Environmental Qualification of Certain Stem Mounted Limit Switches Inside Reactor Containment	2/21/78	All Power Reactor Facilities with an Operating License (OL) or Construction Permit (CP)