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Mr. E. E. Utley
Executive Vice President
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Dear Mr. Utley:

Subject: Request for Additional Information - Shearon Harris

Enclosed are two sets of requests for additional information which were not covered in the Draft Safety Evaluation Report issued in February 1983. Enclosure 1 is the set from the Quality Assurance Branch (Reviewer: Jack Spraul) and Enclosure 2 is the set from the Materials Engineering Branch (Reviewer: David Smith). It would be appropriate to add the listed items to the Open Items List in Chapter I of the DSER. Please provide the information requested by July 1, 1983, or in the alternative, inform us as to those items for which such a schedule cannot be met.

Please contact the Project Manager, Prasad Kadambi, at (301)492-8423 regarding any questions.

Sincerely,

Original signed by:
George W. Knighton

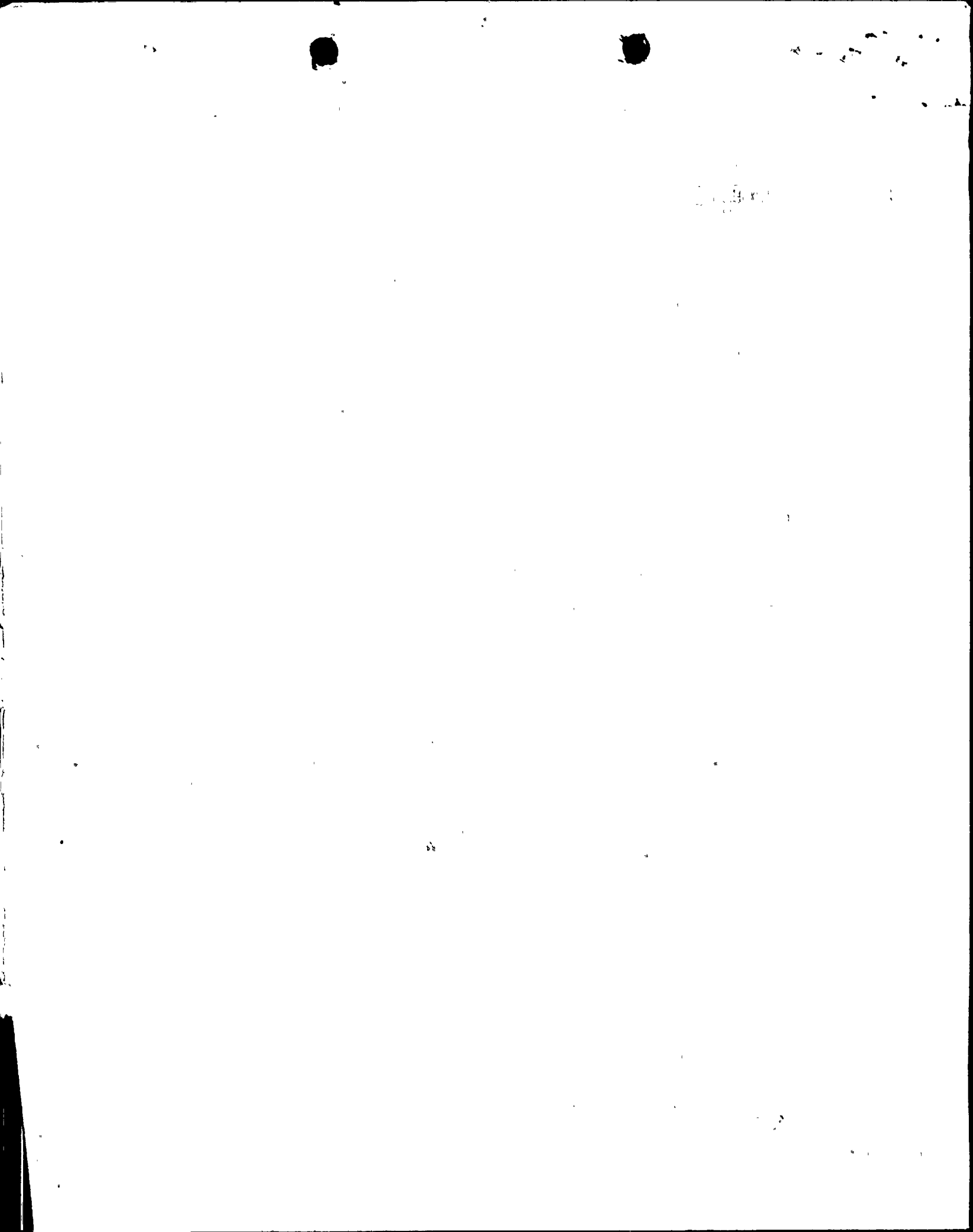
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Enclosures:
As stated

cc: See next page

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ATTACHMENT 1

Harris

Request for Additional Information

260.0 Quality Assurance Branch

260.67 Section 17.1.2.2 of the standard format (Regulatory Guide 1.70) requires the identification of safety-related structures, systems, and components controlled by the QA program. You are requested to supplement and clarify the Harris FSAR in accordance with the following:

- a. The following items do not appear on FSAR Table 3.2.1-1. Add the appropriate items to the table or justify not doing so.
 1. Containment emergency sumps
 2. Equipment hatch and personnel air locks
 3. Guard pipes and leak-tight compartments for containment emergency sump recirculation lines and valves
 4. Engineered safety features actuation system
 5. Charcoal filters in the control room HVAC system
 6. Fuel building radiation monitor
 7. Roof and site drainage system including drains, parapets, grading, culverts, and channels
 8. AC Onsite Power Systems (Class 1E)
 - Diesel generator governor, voltage regulator, and excitation system
 - Instrumentation, control, and power cable splices, connectors, and terminal blocks
 - Conduit and cable trays and their supports which do not contain Class 1E cables but whose failure may damage other safety-related items
 - Load sequencer
 - AC control power inverters
 - AC vital bus distribution equipment
 9. DC Power Systems (Class 1E)

- Conduit and cable trays and their supports which do not contain Class 1E cables but whose failure may damage other safety-related items
 - Battery racks
 - Protective relays and control panels
10. Radioactivity sampling (air, surfaces, liquids)
 11. Radioactivity contamination measurement and analysis equipment
 12. Personnel monitoring equipment (internal, e.g., whole body counts, and external, e.g., TLD system)
 13. Instrument storage, calibration, and maintenance program
 14. Decontamination facilities, personnel, and equipment
 15. Respiratory protection equipment (including testing)
 16. Contamination control
- b. Clarify Table 3.2.1-1 as noted below or justify not doing so.
1. Provide a commitment that all safety-related instrumentation and controls (I&C) described in FSAR Section 7.1 through 7.6 and other safety-related I&C for safety-related fluid systems will be subject to the pertinent requirements of the FSAR Appendix B QA program. This can be done by a footnote to FSAR Table 3.2.1-1.
 2. Under "Electrical Systems and Components" on FSAR Amendment 3 page 3.2.1-43 is "Safety-related power control and instrument cables and raceways." It appears that a comma should be inserted after "power" and one after "control."
 3. Notes 26 and 27 to FSAR Table 3.2.1-1 state that some required information can be found in Table 11.4.2-1. Table 11.4.2-1 does not have this information, and clarification is needed.
- c. FSAR Amendment 3, on page 3.2.1-42, includes "Post Accident Monitoring Instrumentation NUREG 0737." However, Enclosure 2 of NUREG-0737 identified numerous other items that are also safety-related or of such importance to safety that they should have the pertinent requirements of the FSAR Operational QA program applied. These items are listed below. Provide such a commitment in Table 3.2.1-1 of the FSAR or justify not doing so.



11

NUREG-0737
(Enclosure 2)
Clarification Item

1. Plant-safety-parameter display console	I.D.2
2. Reactor coolant system vents	II.B.1
3. Plant shielding	II.B.2
4. Post accident sampling capabilities	II.B.3
5. Valve position indication	II.D.3
6. Auxiliary feedwater system	II.E.1.1
7. Auxiliary feedwater system initiation and flow	II.E.1.2
8. Emergency power for pressurizer heaters	II.E.3.1
9. Dedicated hydrogen penetrations	II.E.4.1
10. Containment isolation dependability	II.E.4.2
11. Instrumentation for detection of inadequate core-cooling	II.F.2
12. Power supplies for pressurizer relief valves, block valves, and level indicators	II.G.1
13. Automatic PORV isolation	II.K.3.1
14. PID controller	II.K.3.9
15. Anticipatory reactor trip on turbine trip	II.K.3.12
16. Power on pump seals	II.K.3.25
17. Emergency plans	III.A.1.1/III.A.2
18. Emergency support facilities	III.A.1.2
19. Inplant I ₂ radiation monitoring	III.D.3.3
20. Control room habitability	III.D.3.4

ATTACHMENT 2

Request for Additional Information

252.0 Materials Engineering Branch - Materials Application Section

252.1 Determine if there was a maximum yield strength specified for austenitic stainless steels, and its magnitude (SRP 4.5.1 and 4.5.2).

252.2 For precipitation hardening and martensitic stainless steels, state the aging and tempering treatments specified (SRP 4.5.1 and 4.5.2).

252.3 Identify the materials used based upon Code Case 1618. If the limitations of Regulatory Guide 1.85 are applicable, justify why the recommendations of the Regulatory Guide were not met (SRP 4.5.2).

252.4 Provide data and rationale as to the acceptability of the electroslag welds in steam generators 1 & 2 (SRP 5.2.3, 5.4.2.1, SRP 1.34, "Control of Electroslag Weld Properties").

- 252.5 There have been many instances of underclad cracking in carbon-manganese steel pressure vessels made without fine grain melt practice or with high heat input weld processes. Provide rationale and data to demonstrate that underclad cracking is not present, and if it is, that it is acceptable (SRP 5.2.3 and 5.3.1, Regulatory Guide 1.43, "Control of Stainless Steel Weld Cladding of Low-Alloy Steel Components").
- 252.6 Provide data and rationale as to the acceptability of the fracture toughness of ferritic steel components in the engineered safety features (SRP 6.1.1).
- 252.7 Provide data and rationale why fracture toughness criteria were applied to Class 2 components of the main steam and feedwater systems and not applied to Class 3 components. Also provide a rationale as to the acceptability of Class 3 components which were not produced to meet a fracture toughness criteria (SRP 10.3.6).
- 252.8 Identify materials by specification, grade, type, etc., rather than by proprietary designations (SRP 4.5.1 and 4.5.2).