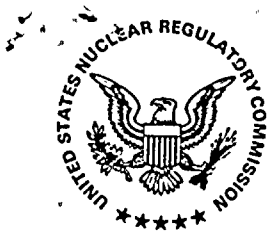


50-400



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 1, 1999

See SER

Mr. James Scarola, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: RELIEF REQUESTS ASSOCIATED WITH SECOND 10-YEAR INTERVAL
INSERVICE TESTING PROGRAM - SHEARON HARRIS NUCLEAR POWER
PLANT, UNIT 1 (TAC NO. MA0815)

Dear Mr. Scarola:

By letter dated January 27, 1998, Carolina Power & Light Company (CP&L) submitted the Shearon Harris Nuclear Power Plant, Unit 1, Inservice Testing (IST) Program for the second 10-year interval, which began on February 2, 1998, and ends on May 1, 2007. That submittal included requests for relief from certain requirements of the 1989 Edition of the ASME (American Society of Mechanical Engineers) Code, Section XI.

The results of the NRC staff's review of these relief requests are documented in the enclosed Safety Evaluation (SE). Attachment 1 to the SE provides a summary of the staff's determination regarding each relief request. Pursuant to 10 CFR 50.55a(f)(6)(i), the staff has determined that with respect to CB-VR1 and IA-VR1, that the requirements of the Code are impractical and relief is granted. The relief granted is authorized by law and will not endanger life or property or the common defense and is otherwise in the public interest giving due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility. Relief requests AF-PR1, CC-VR1, CC-VR2, CE-VR1, CT-VR1, MS-VR1, and SI-VR1 are authorized pursuant to the provisions of 10 CFR 50.55a(a)(3)(ii) in that compliance with the specified requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety. The relief requests that are granted or alternatives authorized by the NRC are acceptable for implementation for the second 10-year interval, with the exception of CE-VR1, which is authorized for one year or until the next refueling outage. Attachment 2 to the SE is the Technical Evaluation Report (TER) provided by the contractor who assisted the staff in this review.

While it was not the staff's intent to complete a detailed review of the IST program, portions of the program were reviewed in the course of evaluating the acceptability of the proposed relief requests. As a result of this effort, the staff identified some programmatic anomalies and issues, for which the staff proposed recommendations. Appendices A and B list the anomalies, issues and recommendations. It is the opinion of the staff that these recommendations be implemented in accordance with the guidance provided. Furthermore, these items should be addressed within one year, or by the end of the next refueling outage, whichever is later.

NRC guidance contained in Generic Letter (GL) 89-04, "Guidance on Developing Acceptable Inservice Testing Programs," provides alternatives to the Code requirements determined to be acceptable to the staff. Alternatives that conform with the guidance in GL 89-04 may be implemented prior to receiving NRC approval, but should be included as relief requests for review by the staff. Consistent with the guidance provided, CP&L submitted relief requests that

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it believes conform to the guidance in GL 89-04. The staff reviewed these requests and the results of its review are summarized in Attachment 1 to the SE.

If you have any questions regarding this matter, please contact Mr. Scott Flanders at (301) 415-1172.

Sincerely,
Original signed by

Cecil O. Thomas, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosure:
Safety Evaluation

cc w/enclosure: See next page

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cc:

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