

CATEGORY 1

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 FACIL:50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina 05000400
 AUTH.NAME AUTHOR AFFILIATION
 VERRILLI,M. Carolina Power & Light Co.
 ROBINSON,W.R. Carolina Power & Light Co.
 RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 97-005-00:on 960320,failure to perform core flux mapping following plant operation with reactor power greater than 100% occurred.Cause was implemented containing inappropriate guidance for short excursions.Change withdrawn.W/970317 ltr.

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NOTES:Application for permit renewal filed. 05000400

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CP&L

Carolina Power & Light Company
PO Box 165
New Hill NC 27562

William R. Robinson
Vice President
Harris Nuclear Plant

MAR 17 1997

U.S. Nuclear Regulatory Commission
ATTN: NRC Document Control Desk
Washington, DC 20555

Serial: HNP-97-069
10CFR50.73

SHEARON HARRIS NUCLEAR POWER PLANT UNIT 1
DOCKET NO. 50-400
LICENSE NO. NPF-63
LICENSEE EVENT REPORT 97-005-00

Sir or Madam:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report describes Technical Specification violation involving the failure to perform flux mapping after short period of plant operation with the reactor greater than 100% power.

Sincerely,

W. R. Robinson

MV

Enclosure

c: Mr. J. B. Brady (HNP Senior NRC Resident)
Mr. L. A. Reyes (NRC Regional Administrator, Region II)
Mr. N. B. Le (NRC - NRR Project Manager)

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Harris Nuclear Plant Unit-1		DOCKET NUMBER (2) 50-400	PAGE (3) 1 OF 2
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TITLE (4)
Failure to perform core flux mapping following plant operation with reactor power greater than 100%.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
3	20	96	97	-- 005	-- 00	3	17	97		
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)									
POWER LEVEL (10) 100%	<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(2)(v)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)						
	<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(x)						
	<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 73.71						
	<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> OTHER						
	<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A						
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)								

LICENSEE CONTACT FOR THIS LER (12)	
NAME Michael Verrilli Sr. Analyst - Licensing	TELEPHONE NUMBER (Include Area Code) (919) 362-2303

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	<input checked="" type="checkbox"/>	NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On March 20, 1996, a calculated reactor power level of 100.08% was identified during the daily secondary calorimetric heat balance. Following verification of the reactor power calculation, reactor power was reduced from approximately 99.9% to 99.5% indicated power. Technical Specification (TS) 3.2.3 action C.1 requires reactor core flux mapping to be performed within 24 hours of initially entering the "prohibited region of operation." This region is defined by TS figure 3.2-3, which has a vertical line at exactly 100.0% power. Flux mapping was not performed following this event based on guidance contained in the bases for TS 3.2.3, which was inserted following a previous violation of this technical specification (reference LER #96-03). This guidance was intended to clarify when thermal power is considered to be within 100.0% rated thermal power and was implemented for the pending continuous calorimetric capability. However, following additional review and delays in implementing the continuous calorimetric, the TS bases revision was considered inappropriate. On February 13, 1997, the bases revision was withdrawn, thereby resulting in the March 20, 1996 event being considered reportable as a violation of TS 3.2.3 for not performing core flux mapping.

This event was caused when the bases change for TS 3.2.3 was implemented containing inappropriate guidance for short power excursions over 100.0% power. (reference CP&L letter to NRC dated February 8, 1996)

Corrective actions included withdrawing the TS 3.2.3 bases change and issuing an Operations Night Order communicating this action to Operations personnel.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Shearon Harris Nuclear Plant - Unit #1	50-400	97	005	00	2 OF 2

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION:

On March 20, 1996, a calculated reactor power level of 100.08% was identified during the daily secondary calorimetric heat balance (Operations Surveillance Test, OST-1004). Following verification of the reactor power calculation, reactor power was reduced from approximately 99.9% to 99.5% indicated power. Technical Specification 3.2.3, Heat Flux Hot Channel Factor and RCS Flow Rate and Nuclear Enthalpy Rise Hot Channel Factor, action C.1 requires reactor core flux mapping to be performed within 24 hours of initially entering the "prohibited region of operation." This region is defined by TS figure 3.2-3, which has a vertical line at exactly 100.0% power. Flux mapping was not performed following this event based on guidance contained in the bases for TS 3.2.3, which was inserted following a previous violation of this technical specification. (reference LER #96-03) This guidance was intended to clarify when thermal power is considered to be within 100.0% rated thermal power and was implemented for the pending continuous calorimetric capability. However, following additional review and delays in implementing the continuous calorimetric, the TS bases revision was considered inappropriate. On February 13, 1997, by CP&L letter to the NRC, the bases revision was withdrawn, thereby resulting in the March 20, 1996 event being considered reportable as a violation of TS 3.2.3 for not performing core flux mapping.

CAUSE:

This event was caused when the bases change for TS 3.2.3 was implemented containing inappropriate guidance for short power excursions over 100.0% power. (reference CP&L letter to NRC dated February 8, 1996)

SAFETY SIGNIFICANCE:

There were no safety consequences as a result of this event. Short periods of reactor power less than 102% rated thermal power are accounted for in accident and hot channel factor analyses.

This is being reported per 10CFR50.73.a.2.i.B as a violation of Technical Specifications.

PREVIOUS SIMILAR EVENTS:

LER 96-003 was submitted on February 23, 1996.

CORRECTIVE ACTIONS COMPLETED:

1. The TS 3.2.3 bases change was withdrawn by letter to the NRC dated February 13, 1997, thereby deleting the intended clarification of 100.0% rated thermal power.
2. An Operations Night Order was issued on February 5, 1997 to inform Operations personnel of the intent to delete the TS bases guidance for 100% rated thermal power.



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