

# CATEGORY 1

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SUBJECT: Proposes implementing change to Bases for TSs 3/4.6.1.4 & 3/4.6.1.6 to change calculated peak pressure for MSLB event.

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United States Nuclear Regulatory Commission  
ATTENTION: Document Control Desk  
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**SHEARON HARRIS NUCLEAR POWER PLANT  
DOCKET NO. 50-400/LICENSE NO. NPF-63  
TECHNICAL SPECIFICATION BASES CHANGE - CONTAINMENT ANALYSIS**

Gentlemen:

In accordance with 10 CFR 50.36(a), the Harris Nuclear Plant is proposing to implement a change to the Bases for Technical Specifications (TS) 3/4.6.1.4 and 3/4.6.1.6.

In response to NRC Information Notice 87-65, "Plant Operation Beyond Analyzed Conditions," an analysis was performed to evaluate the temperature distribution inside containment. That analysis concluded that the containment temperature is normally stratified, and under worst case conditions, would result in an average temperature of 135°F if thoroughly mixed. Containment temperature and pressure for the limiting MSLB and LOCA scenarios were recalculated using 135°F as the pre-accident temperature. 120°F had been used as the pre-accident temperature in previous analyses.

The revised analyses show that the peak temperature and pressure values remain within the accident acceptance criteria. The peak containment pressure following the most limiting LOCA scenario is essentially unchanged and remains below the peak accident pressure, Pa, as stated in TS 3.6.1.2 and the corresponding Bases. Therefore, offsite dose projections are not affected and there will be no increase in the consequences of this scenario.

However, the change in calculated peak pressure for the MSLB event will require revision of the Bases for Internal Pressure (TS 3.6.1.4) and Containment Structural Integrity (TS 3.6.1.6). Specifically, the peak pressure expected from a postulated MSLB event is now 41.2 psig, which is an increase above the current Bases peak values of 40.9 psig (TS 3.6.1.4) and 41 psig (TS 3.6.1.6). However, the revised peak pressure of 41.2 psig remains below the maximum internal design pressure of 45 psig, as stated in TS 5.2.2, Design Pressure and Temperature.

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The attached retyped Bases pages B 3/4 6-1 and B 3/4 6-2 are provided for your information.

Questions regarding this matter may be referred to Mr. T. D. Walt at (919) 362-2711.

Sincerely,



Enclosure

LSR/lsr

c: Mr. J. B. Brady-NRC Resident Inspector  
Mr. S. D. Ebnetter-NRC Regional Administrator  
Mr. N. B. Le-NRC Project Manager