

.



REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

FACIL:50 AUTH.NA	AME AUTHOR	iling Water F AFFILIATION	5/04/20 NOTARIZED Reactor, Dairyland		DOCKET # 05000409	
GREEN,W BERG,W.J RECIP.J	L. Dairylan	d Power Coope d Power Coope NT AFFILIATIC	erative			P'
SUBJECT	: LER 95-001-00:on	950412,noted	l that HPSW to cont	ainment	_	R
	test of 1A EDG.P	rocedures for	omentary LOP due t test being modifi being open.W/95042	ed to incl	onal Lude	I
DISTRIB	UTION CODE: IE22T	- COPIES RECEI	VED:LTR / ENCL /	SIZE:	3	0
	-	_	t (LER), Incident		05000409	R
Co	onversion App 5/25	/72. FCLB/Ga	skin,C. 1cy.	/20//0/.	0000400	Ι
	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENG	זי.	Т
	ONDD PD		FAIRTILE, M			Y
	AEOD/SPD/RAB FILE_CENTER NRR/DE/EELB NRR/DISP/PIPB NRR/DRCH/HHFB	2 2 1 1 1 1 1 1 1 1 1 1 1 1	AEOD/SPD/RRAB NRR/DE/ECGB NRR/DE/EMEB NRR/DOPS/OECB NRR/DRCH/HICB		·	1
	NRR/DRCH/HOLB NRR/DSSA/SPSB/B		NRR/DSSA/SPLB NRR/DSSA/SRXB	1 1 1 1 1 1	•	D
EXTERNAL:	RES/DSIR/EIB L ST LOBBY WARD	1 1	RGN3 FILE 01 LITCO BRYCE,J H	2 2		0
	NOAC MURPHY,G.A NRC PDR	1 1 1 1	NOAC POORE,W. NUDOCS FULL TXT	1 1 1 1	ł	C,
NOTES:		1 1	م ۱			: U
	,		4			М
						Е
						Ν
						т
PL DE	TO ALL "RIDS" RECIPIENTS; EASE HELP US TO REDUCE SK, ROOM P1-37 (EXT. 504-200 STRIBUTION LISTS FOR DO	33) TO ELIMINATE Y	THE DOCUMENT CONTROL OUR NAME FROM OT NEED!			

FULL TEXT CONVERSION REQUIRED TOTAL NUMBER OF COPIES REQUIRED: LTTR 27 ENCL 27 COOPERATIVE . 3200 EAST AVE. SO. . P.O. BOX 817 . LA CROSSE, WISCONSIN 54602-0817

(608) 788-4000 FAX NO. (608) 787-1420

WILLIAM L. BERG General Manager

April 20, 1995

In reply, please refer to LAC-13480

DOCKET NO. 50-409

Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555

DAIRYLAND

Gentlemen:

SUBJECT: Dairyland Power Cooperative La Crosse Boiling Water Reactor (LACBWR) Possession-Only License No. DPR-45 Licensee Event Report No. 95-001

REFERENCES: (1) 10 CFR 50.73

In accordance with 10 CFR 50.73, attached is Licensee Event Report No. 95-001.

If there are any questions, please contact us.

Sincerely,

DAIRYLAND POWER COOPERATIVE

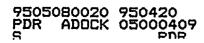
William L Berg

William L. Berg, General Manager

WLB:WRG:dh

cc: John Martin, Regional Administrator U. S. Nuclear Regulatory Commission, Region III

> Morton Fairtile, Project Manager Office of Nuclear Reactor Regulation U. S. Nuclear Regulatory Commission



•																
NRC Form (9-83)	366				LIC	ENSEE	EVE	NT RE	PORT	(LER)	4	APPF	R REGUL ROVED ON RES: 8/31/	AB NO. 3		
FACILITY	NAME (1	. .					``	·····			DOCKET NUMBER	(2)			PAGE	(3)
		-	TI ING	WATER REA	ACTOR (1	ACRWR	1				0 15 10 10	0	4101	9 1	OF	012
TITLE (4)			16110				·/				<u> </u>			- 1		<u> </u>
LOSS	5 OF (CONTA	AINME	NT BUILDIN	G BASEM	ENT F	IRE H	IOSE S	TATIO	N OPERAB	ILITY					
EVE	NT DATE	(5)	Г <u> </u>	LER NUMBER (6	5	REP	ORT DAT	E (7)		OTHE	R FACILITIES INVO	LVED	(8)			
MONTH	DAY YEAR YEAR SEQUENTIAL REVISION			MONTH	DAY	YEAR	EAR FACILITY NAMES			000	KET NUM	BER(S)				
			i — —									0	50	0 0		
	.		Į .					. ſ								
04	12	95	9 5	001	00	04	1 7	9 5				0	5 0	0 0		
	RATING		THIS RE	PORT IS SUBMITTER	D PURSUANT 1	O THE RE	QUIREME	NTS OF 10	CFR §: /(Check one or more	of the following) (1	1)	r -			
MO	DE (9)	<u> </u>	20.	.402(b)		20.405(c)			50,73(s)(2)(iv)		L	73,71(b)			
POWER		~ ~	{	.405(=){1}(i)		50,35(c)				50,73(a)(2)(v)			73.71(c)			
(10)	<u> </u>	0 10		405(s){1){#}		50.36(c)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form				
			20.405(s)(1)(iii) X 50.73(s)(2)(i) 50.73(s)(2)(viii)(A) 366A)													
				.405(a)(1)(lv)		60.73(a) 60.73(a)				50,73(s)(2)(viii) 50,73(s)(2)(x)	(8)					
				.405(s)(1)(v)				FOR THIS	1 68 (12)	00,73(87(2)(4)		I		<u>.</u>		
NAME								ron mis				TEL	EPHONE N	UMBER		
WILL	IAM R	GR	EEN,	TECHNICAL	SUPPORT	ENGI	NEER	•				6	8 <mark>9</mark>	- 14	121	1,0
				COMPLETE	ONE LINE FOR	EACH CO	MPONENT	FAILURE	DESCRIBE	D IN THIS REPO	DRT (13)				J	
CAUSE	SYSTEM	сомр	ONENT	MANUFAC- TURER	REPORTABLE TO NPRDS			CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	RI	EPORTABL	.E		
А			1 1	1 1 1	N											
	-	1	1 1													
				SUPPLEME	NTAL REPORT	EXPECTE	D (14)				EXPECT	ED	мо	VTH C	AY	YEAR
I				SUBMISSION DATE		X	NO				SUBMISSI DATE (1	ON				

BSTRACT (Limit to 1400 speces, i.e., approximately litteen single-spece typewritten lines) (10)

The La Crosse Boiling Water Reactor (LACBWR) Technical Specifications (T.S.) 4.4.5 states, "The fire hose stations in the following locations shall be operable." This statement is followed by a list of five stations that includes the "Containment Building Basement." This requirement is applicable at all times. The action step of this T.S. states, "With a hose station inoperable, establish a 1-hour fire watch, or route an additional hose of equivalent capacity to the unprotected area within one hour." On April 12, 1995, it was noted by a LACBWR staff member that the High Pressure Service Water to the Containment Building, which supplies water pressure to the Containment Building Basement hose station, was isolated.

NRC Form 366A (9-83) LICENSEE EVENT	REPORT (LER) TEXT CONTIN	U.S. NUCLEAR REGULATORY COMMISSION NUATION APPROVED OMB NO, 3150-0104 EXPIRES: 8/31/88
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
		YEAR SEQUENTIAL REVISION NUMBER
LA CROSSE BOILING WATER REACTOR	0 15 10 10 10 14 10 19	9 915 - 0101 - 010 012 05 012

The La Crosse Boiling Water Reactor (LACBWR) Technical Specifications (T.S.) require that several fire hose stations be operable at all times. Included in the list of hose stations is the Containment Building Basement hose station.

On the morning of Wednesday, April 12, 1995, a LACBWR operator discovered the High Pressure Service Water (HPSW) Containment Building automatic isolation valve shut, which rendered the Containment Building hose station inoperable. At the time of this discovery, the operators on duty were unsure of when the isolation had occurred, or what caused the isolation. The shift supervisor was informed and the valve was opened.

Further investigation of the incident revealed that the valve had probably closed on the morning of April 10, 1995. On this date we had performed an operational test of the 1A Emergency Diesel Generator (EDG) which caused momentary loss of power to the 120 VAC regulated bus during the Power Supply transfer. This caused a momentary loss of control power to the HPSW auto isolation valve and caused the valve to fail closed. This condition went unnoticed until April 12, 1995. Portions of the operational test of the 1A EDG were repeated on April 18, 1995, to confirm our analysis of this incident.

The effect of this situation in Plant Safety was minimal. Numerous fire extinguishers are located throughout the Containment Building to be used in the event of a fire. The HPSW auto isolation valve had the capability of being unisolated in the event of a fire and routine tours (every 4 hours) are conducted in the Containment Building to identify possible hazardous conditions such as fire hazards.

To ensure this situation doesn't occur again, the procedure for the operational test of the 1A EDG that was performed on April 10, 1995, is being modified to include a statement that instructs operators to verify open, and reopen as necessary, the HPSW auto isolation valve during this procedure.

. . 7 • ĸ

• . . · · · ·

`

· · · ·

,

1