

Response to NRC Request for Data Verification

B&W Nuclear Service Company
Report BAW-2154, March 1992
Analysis of Capsule V
Carolina Power & Light Company
Shearon Harris Unit No.1

-- Reactor Vessel Material Surveillance Program --

Corrected Page 7-14

9406170132 940610
PDR ADCK 05000400
P PDR



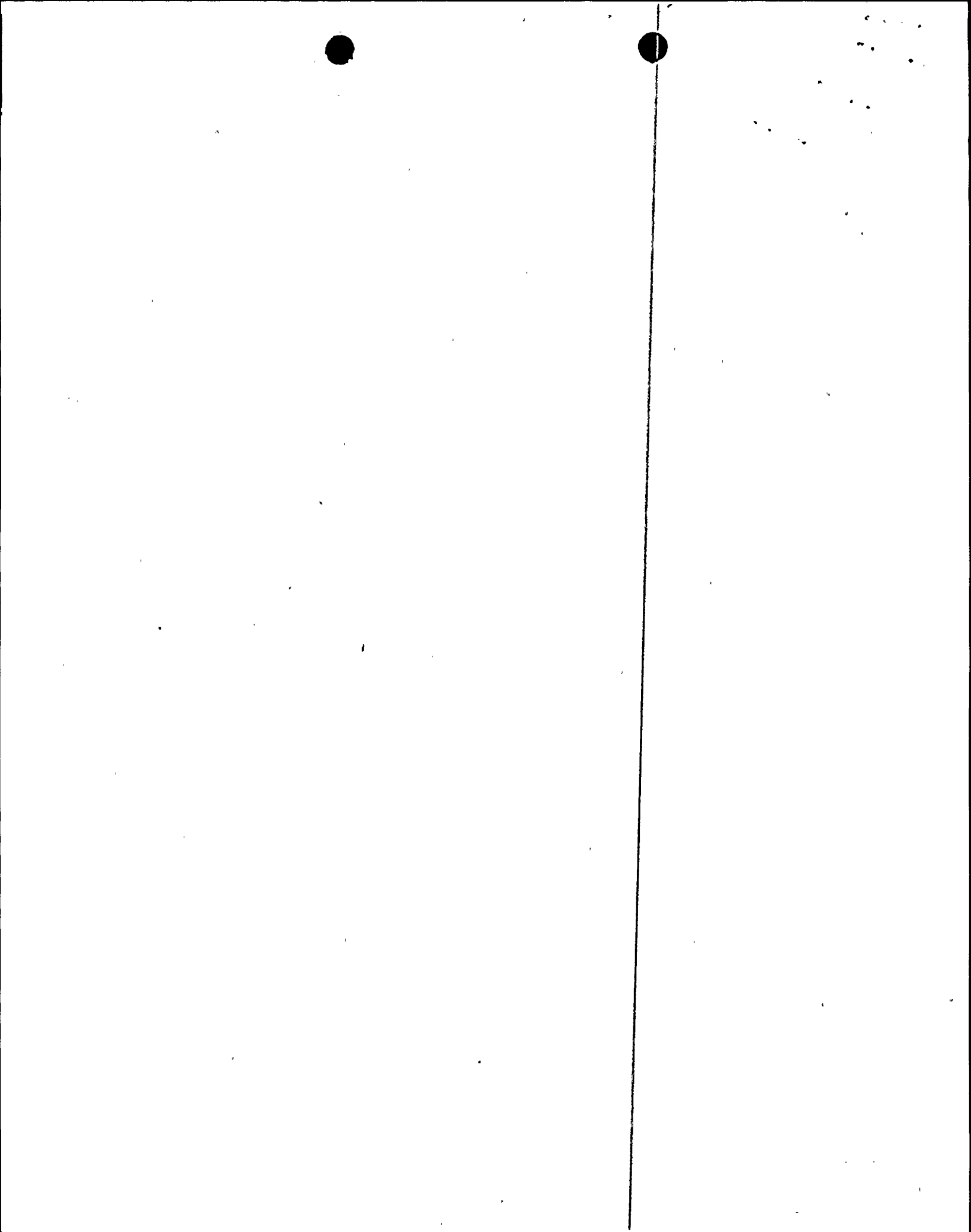


Table 7-7. Evaluation of Reactor Vessel End-of-Life Upper Shelf Energy - Shearon Harris Unit 1

Material Description			Material Chemical Composition, w/o ^(a)		Estimated EOL Fluence		Initial USE ^(d) ft-lbs	Estimated EOL-USE ^(b) Per R.G. 1.99/2 ^(b)		Estimated EFPY to 50 ft-lbs	
Reactor Vessel Beltline Region Location	Heat Number	Type	Copper	Nickel	Inside Surface n/cm ^{2(c)}	T/4 Wall Location n/cm ^{2(c)}		Inside Surface	T/4 Wall Location	I.S.	T/4
Intermediate Shell	A9153-1	SA533, GrB1	0.09	0.45	3.42E+19	2.15E+19	83	62 53	64 56	>32	>32
Intermediate Shell	B4197-2	SA533, GrB1	0.10	0.50	3.42E+19	2.15E+19	71 ^(d)	53	55	>32	>32
Lower Shell	C9924-1	SA533, GrB1	0.08	0.45	3.42E+19	2.15E+19	98	73 76	76 79	>32	>32
Lower Shell	C9924-2	SA533, GrB1	0.08	0.44	3.42E+19	2.15E+19	88	66 59	68 71	>32	>32
Inter. to Lower Shell Weld	AB	ASA/Linde 124	0.04	0.95	3.42E+19	2.15E+19	88	66 51	68 70	>32	>32
Interm. Longit. Welds (Both)	BC/BD	ASA/Linde 124	0.06	0.91	1.33E+19	8.36E+18	94	74	76	>32	>32
Lower Longit. Welds (Both)	BA/BB	ASA/Linde 124	0.06	0.91	1.33E+19	8.36E+18	94	74	76	>32	>32

^(a)All data per WCAP-10502, May 1984

^(b)Upper-shelf values calculated per Regulatory Guide 1.99, Revision 2, dated May, 1988

^(c)Section 6, Table 6-3 (this report)

^(d)Revised initial upper-shelf energy value - see Paragraph 7.4

^(e)Calculated per Regulatory Guide 1.99, Revision 2, dated May 1988.



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