ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATOR INFORMATION DISTRIBUTION STATEM (RIDS)

ACCESSION NBR:9212010016 DOC.DATE: 92/12/01 NOTARIZED: NO DOCKET # FACIL:50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina 05000400 AUTH.NAME AUTHOR AFFILIATION STARKEY, R.B. Carolina Power & Light Co. RECIP.NAME RECIPIENT AFFILIATION Document Control Branch (Document Control Desk) R SUBJECT: Clarifies 920116 response to NRC station blackout SE Sections 2.2 & 2.2.5. New procedure will require that battery room temp be verified at not less than 70 F once per day. Valve 2MD-V77SB-1 will be manually closed. D DISTRIBUTION CODE: A050D COPIES RECEIVED:LTR (ENCL TITLE: OR Submittal: Station Blackout (USI A-44) 10CFR50.63, MPA A-22 NOTES: Application for permit renewal filed. 05000400 RECIPIENT COPIES RECIPIENT COPIES ID CODE/NAME LTTR ENCL ID CODE/NAME LTTR ENCL PD2-1 PD 1 LE,N INTERNAL: ACRS AEOD/DSP/TPAB NRR PD2-4PM TAM 1 NRR/DET/ESGB 8D NRR/DST/SELB NRR/DST/SPLB8D1 REG_FILE NRR/DST/SRXB8E 1 01 1 EXTERNAL: NRC PDR 1 NSIC

NOTE TO ALL "RIDS" RECIPIENTS:

PLEASE HELP US TO REDUCE WASTE! CONTACT THE DOCUMENT CONTROL DESK. ROOM P1-37 (EXT. 504-2065) TO ELIMINATE YOUR NAME FROM DISTRIBUTION LISTS FOR DOCUMENTS YOU DON'T NEED!

4 И • * il , a CP&L

Carolina Power & Light Company

P.O. Box 1551 • Raleigh, N.C. 27602

SERIAL: NLS-92-152 10 CFR 50.63

R. B. STARKEY, JR. Vice President Nuclear Services Department

United States Nuclear Regulatory Commission ATTENTION: Document Control Desk Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT DOCKET NO. 50-400/LICENSE NO. NPF-63

CLARIFICATIONS ON STATION BLACKOUT

Gentlemen:

On January 16, 1992, Carolina Power & Light Company (CP&L) responded to the NRC's Station Blackout Safety Evaluation (SE) for the Shearon Harris Nuclear Power Plant (SHNPP). The purpose of this letter is to provide clarifications to that response pertaining to SE Sections 2.2 and 2.2.5.

Carolina Power & Light Company's response to SE Section 2.2 stated that an existing procedure would be modified to verify that the battery room temperature is not less than 70°F. We have subsequently determined that it is more appropriate to create a new procedure, an Operations Performance Test, for this purpose. The new procedure will require that the battery room temperature be verified as not less than 70°F once per day.

In response to Section 2.2.5, we stated that Valve 2MD-V77SB-1 would be utilized for containment isolation. However, there is an inboard containment isolation valve, 2MD-V36SA-1, which may also be used to ensure containment isolation. Therefore, Valve 2MD-V77SB-1 will be manually closed to provide containment isolation of the containment sump discharge through penetration X-74 if containment isolation is required and it cannot be verified from Control Room indication that the penetration is already isolated by at least one valve.

Should you have any questions regarding these clarifications, please contact Mr. R. W. Prunty at (919) 546-7318.

Yours very truly,

R. B. Starkey, Jr.

LSR/jbw

cc: Mr. S. D. Ebneter

Mr. N. B. Le

Mr. J. E. Tedrow

9212010014 921201 PDR ADDCK 05000400 P PDR A050 (1000)

ė • •