

ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9208170190 DOC. DATE: 92/08/14 NOTARIZED: NO DOCKET #
 FACIL: 50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina 05000400
 AUTH. NAME AUTHOR AFFILIATION
 VERRILLI, M. Carolina Power & Light Co.
 HINNANT, C. S. Carolina Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 92-009-00: on 920715, manual reactor trip/AFW actuation occurred due to low SG level. Caused by inadvertent deenergization of MFP recirculation valve power supply. Supply breaker replaced. W/920814 ltr.

DISTRIBUTION CODE: IE22T COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 30.73/30.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: Application for permit renewal filed. 05000400

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	PD2-1 LA		1	1		PD2-1 PD		1	1
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INTERNAL:	ACNW		2	2		ACRS		2	2
	AEOD/DOA		1	1		AEOD/DSP/TPAB		1	1
	AEOD/ROAB/DSP		2	2		NRR/DET/EMEB 7E		1	1
	NRR/DLPQ/LHFB10		1	1		NRR/DLPQ/LPEB10		1	1
	NRR/DOEA/GEAB		1	1		NRR/DREP/PRPB11		2	2
	NRR/DST/SELB 8D		1	1		NRR/DST/SICBSH3		1	1
	NRR/DST/SPLBSD1		1	1		NRR/DST/SRXB 8E		1	1
	<u>REG FILE</u> 02		1	1		RES/DSIR/EIB		1	1
	RGN2 FILE 01		1	1					
EXTERNAL:	EG&G BRYCE, J. H		2	2		L ST LOBBY WARD		1	1
	NRC PDR		1	1		NSIC MURPHY, G. A		1	1
	NSIC POORE, W.		1	1		NUDOCS FULL TXT		1	1

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Carolina Power & Light Company

P.O. Box 165 • New Hill, N.C. 27562

C. S. HINNANT
General Manager - Harris Plant

AUG 14 1992

Letter Number: HO-920119

U.S. Nuclear Regulatory Commission
ATTN: NRC Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT UNIT 1
DOCKET NO. 50-400
LICENSE NO. NPF-63
LICENSEE EVENT REPORT 92-009-00

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours

C. S. Hinnant
General Manager
Harris Nuclear Project

MV:dmw

Enclosure

cc: Mr. S. D. Ebnetter (NRC - RII)
Mr. N. B. Le (NRC - RII)
Mr. J. E. Tedrow (NRC - SHNPP)
Mr. G. E. Vaughn

170049

MEM/LER92-009/1/OS1

9208170190 920814
PDR ADCK 05000400
S PDR

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) **Shearon Harris Nuclear Power Plant - Unit #1** DOCKET NUMBER (2) **0 5 0 0 0 4 0 0** PAGE (3) **1 OF 0 3**

TITLE (4) **Manual Reactor Trip/AFW Actuation due to Low S/G level caused by inadvertently deenergizing MFP recirc valve power supply.**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)											
0	7	1	5	9	2	9	2	0	0	9	0	0	0	0	0	0	0	0	0	0	0	0

OPERATING MODE (9) **1**

POWER LEVEL (10) **3 10**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **Michael Verrilli Specialist - Regulatory Compliance** TELEPHONE NUMBER **9 1 9 3 6 2 - 1 2 3 0 1 3**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	E, B	B, K, R	B 4 5 5	Y					

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT:

On July 15, 1992 a supply breaker failure occurred while starting Exhaust Fan E-18 which resulted in the fan running with both the inlet and outlet dampers closed. This condition created a potential for fire in the fans charcoal filter unit. Remote and local efforts were made to trip open the breaker and secure the fan, but were unsuccessful. The control room staff determined that the supply bus for the E-18 fan (Bus #1D-2) would have to be deenergized to allow the fan breaker to be racked out and the fan secured. When 1D-2 was deenergized it caused the "B" Main Feed Pump and "B" Condensate Booster Pump Recirculation Valves to fail to the full-open position. This caused the running Main Feed Pump to trip on low suction pressure and resulted in rapidly decreasing Steam Generator water levels. A manual reactor trip was initiated by the control operator as required. All safety systems functioned as required including an automatic Auxiliary Feedwater System actuation. The cause of this event was a failure of the E-18 supply breaker. This breaker was replaced and satisfactorily tested on July 15, 1992.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Shearon Harris Nuclear Power Plant Unit #1	DOCKET NUMBER (2) 0 5 0 0 0 4 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR 92	SEQUENTIAL NUMBER - 0 6	REVISION NUMBER 9 - 0 0	0	2	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION:

On July 15, 1992 at approximately 0230, an attempt was made from the main control room to start Reactor Auxiliary Building Exhaust Fan E-18. This start attempt resulted in no "run" indication or apparent control of the fan. Auxiliary operators were sent to the fan and fan supply breaker to investigate. They observed that the fan was running and that the inlet and outlet dampers were both closed. Attempts to secure the fan both locally and from the control room were unsuccessful. This situation created an immediate concern over the possibility of a fire in the fans charcoal filter unit. The control room staff decided to deenergize the supply bus (Bus #1D-2) for the E-18 fan to prevent this possibility. After completing a review to identify any loads that would be affected by deenergizing bus 1D-2, the bus supply feeder was opened and the supply breaker for E-18 was racked out. Shortly after 1D-2 was deenergized, control room operators observed "B" Main Feed Pump suction pressure decreasing. Attempts were immediately made to increase condensate booster pump speed, but they did not prevent the feed pump from tripping on low suction pressure. A manual reactor trip and turbine trip was then initiated as required, due to rapidly decreasing steam generator levels. All safety systems operated as required including an automatic Auxiliary Feedwater actuation due to the loss of the running main feed pump. The plant was stabilized with the appropriate emergency operating procedures in Mode-3.

This event is being reported in accordance with 10CFR50.73(a)(2)(iv) as an Engineered Safety Feature and Reactor Protection System actuation. There have been no similar events reported.

CAUSE:

The cause of this event was a failure of the E-18 supply breaker. This failure prevented the breaker from being tripped both manually and electrically and also blocked the open signal to the inlet and outlet dampers from the control switch. A contributing factor was that the "B" Main Feed Pump and "B" Condensate Booster Pump Recirc Valves were not identified during the load review conducted prior to deenergizing 1D-2. During normal operating conditions, more time would have been available to fully research affected loads and take the needed precautions, but due to the potential fire threat, review time was limited and this component was not identified.

SAFETY SIGNIFICANCE:

There were no safety consequences as a result of this event. All safety systems functioned as required to place the plant in a safe condition and the possibility of a fire in the E-18 charcoal filter was averted by deenergizing bus 1D-2.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Shearon Harris Nuclear Power Plant Unit #1	DOCKET NUMBER (2) 0 5 0 0 0 4 0 0	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 2	- 0 0 9	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTIONS:

1. The supply breaker for the E-18 Fan was replaced on July 15, 1992.
2. Investigation into the breaker failure revealed mechanical binding in the breakers arcing contact. The apparent cause for this binding was pitting on the arcing contact spring retaining pin. As a result of this, Preventive Maintenance Procedure PM-E0012 was revised to include periodic inspections of this pin and buffing or pin replacement if needed.
3. Training will be performed for operations personnel to ensure their awareness of this event.

EIIS INFORMATION:

Medium Voltage Power System (Class-1E) - EB