

# ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9102080132      DOC. DATE: 91/02/04      NOTARIZED: NO      DOCKET #  
 FACIL: 50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina      05000400  
 AUTH. NAME      AUTHOR AFFILIATION  
 VERRILLI, M.      Carolina Power & Light Co.  
 RICHEY, R. B.      Carolina Power & Light Co.  
 RECIP. NAME      RECIPIENT AFFILIATION

SUBJECT: LER 90-001-00: on 910103, operations personnel identified procedural deficiency that resulted in check valves not being tested as required. Caused by procedural deficiency. Procedure revised to incorporate testing. W/910204 ltr.

DISTRIBUTION CODE: IE22T      COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 3  
 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: Application for permit renewal filed. 05000400

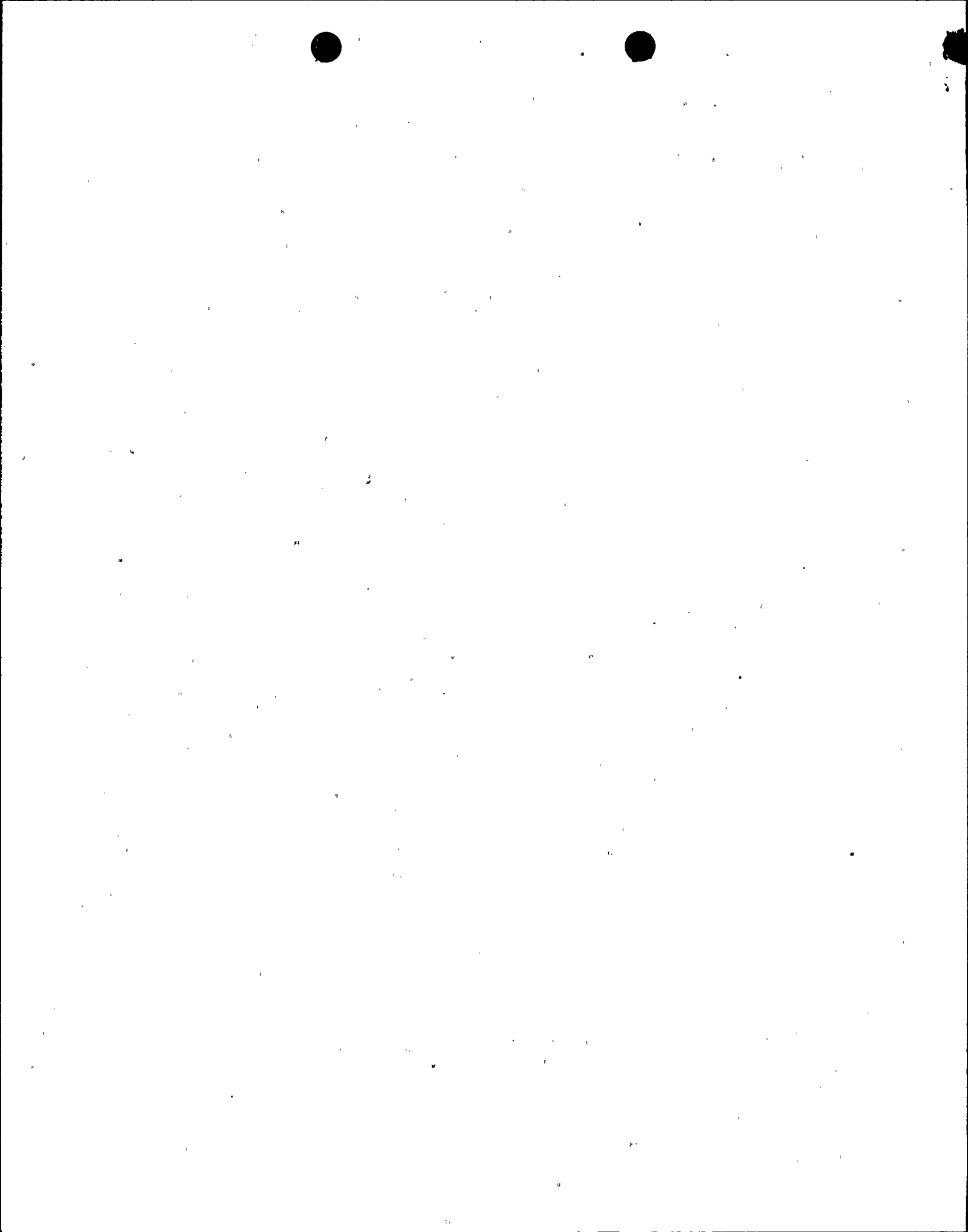
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INTERNAL:	ACNW	2 2	ACRS	2 2
	AEOD/DOA	1 1	AEOD/DSP/TPAB	1 1
	AEOD/ROAB/DSP	2 2	NRR/DET/ECMB 9H	1 1
	NRR/DET/EMEB 7E	1 1	NRR/DLPQ/LHFB11	1 1
	NRR/DLPQ/LPEB10	1 1	NRR/DOEA/OEAB	1 1
	NRR/DREP/PRPB11	2 2	NRR/DST/SELB 8D	1 1
	NRR/DST/SICB 7E	1 1	NRR/DST/SPLB8D1	1 1
	NRR/DST/SRXB 8E	1 1	REG FILE 02	1 1
	RES/DSIR/EIB	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G BRYCE, J. H	3 3	L ST LOBBY WARD	1 1
	NRC PDR	1 1	NSIC MAYS, G	1 1
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**CP&L**

**Carolina Power & Light Company**

P. O. Box 165 • New Hill, N. C. 27562

**HARRIS NUCLEAR PROJECT**

P. O. Box 165  
New Hill, NC 27562

R. B. RICHEY  
Vice President  
Harris Nuclear Project

FEB 0 4 1991

Letter Number: HO-910016 (O)

U.S. Nuclear Regulatory Commission  
ATTN: NRC Document Control Desk  
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT UNIT 1  
DOCKET NO. 50-400  
LICENSE NO. NPF-63  
LICENSEE EVENT REPORT 91-001-00

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,



R. B. Richey  
Vice President  
Harris Nuclear Project

RBR:gcm

Enclosure

cc: Mr. R. A. Becker (NRR)  
Mr. S. D. Ebnetter (NRC - RII)  
Mr. J. E. Tedrow (NRC - SHNPP)

MEM/HO-9100160/1/OS1

9102080132 910204  
PDR ADOCK 05000400  
S

*Handwritten initials and date:*  
JED  
1/1

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) <b>Shearon Harris Nuclear Plant Unit #1</b>	DOCKET NUMBER (2) <b>0 5 0 0 0 4 0 0</b>	PAGE (3) <b>1 OF 2</b>
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TITLE (4)  
**Technical Specification violation due to missed surveillance testing for Control Room HVAC check valves. (TST program/procedure deficiency)**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
01	03	91	91	001	0	02	04	91			
									DOCKET NUMBER(S) <b>0 5 0 0 0</b>		

OPERATING MODE (9) <b>1</b>	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) <b>1, 0, 0</b>	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.38(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.38(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)	
NAME <b>Michael Verrilli, Specialist - Reg. Compliance</b>	TELEPHONE NUMBER <b>9 1 9 3 6 2 - 2 3 0 3</b>

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS		

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO					

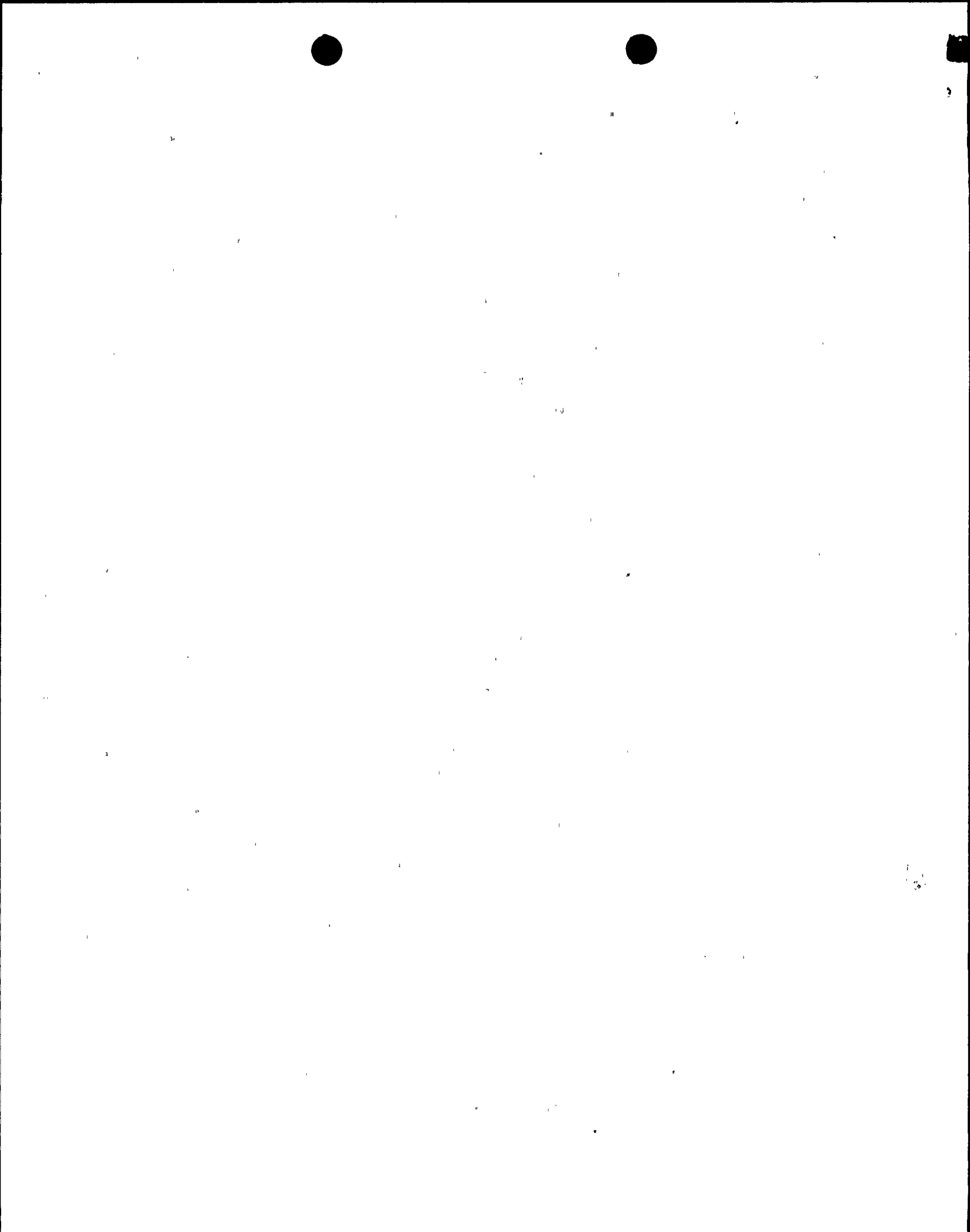
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 1/3/91 the plant was operating in Mode-1 at 100 percent power. During a procedure review of OST-1131 (Control Room Area HVAC System Inservice Test - Quarterly Interval), Operations personnel identified a procedural deficiency that resulted in two check valves not being tested as required. Inservice Inspection Test (ISI-203) implements the requirements of Technical Specification (TS) Section 4.0.5, which in turn implements ASME Section XI testing. Check valves #1CZ-15 and 1CZ-16 are required to be full forward flow tested, per ISI-203, on a quarterly basis. During the aforementioned review of OST-1131, it was revealed that the acceptance criteria of the procedure erroneously took credit for testing these check valves, but in actuality, did not conduct or document an independent forward flow test for these check valves.

The cause of this event was procedural deficiency. OST-1131 has never included steps to properly test these two check valves. This procedure was immediately revised to incorporate the required testing and was successfully performed on 1/18/91.

There were no significant safety consequences as a result of this procedural deficiency. This is based on the following evidence that validates the functional status of these valves and the fact that the systems safety function has never been challenged by radiological conditions. Forward flow is tested through at least one of these check valves once per 18 months by the performance of OST-1231 (Control Room Emergency Filtration Operability Test). This test has consistently verified forward flow and was last performed satisfactorily on 10/1/90. This provides a high degree of confidence that the valves would have functioned as designed if needed.

This is being reported in accordance with 10CFR50.73 (a) (2) (i) (B) as a TS violation.



LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 500 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Shearon Harris Nuclear Plant  Unit #1	DOCKET NUMBER (2)  0   5   0   0   0   4   0   0	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9   1	-   0   0   1	-   0   0	0   2	OF 0   2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT DESCRIPTION:

On 1/3/91 the plant was operating in Mode-1 at 100 percent power. During a procedure review of OST-1131, Operations personnel identified a procedural deficiency that resulted in two check valves not being tested as required. Inservice Inspection Test (ISI-203) implements the requirements of TS Section 4.0.5, which in turn implements ASME Section XI testing. Check valves #ICZ-15 and ICZ-16 are required to be full forward flow tested, per ISI-203, on a quarterly basis. During the aforementioned review of OST-1131, it was revealed that the acceptance criteria of the procedure did in fact take credit for testing these check valves, but in actuality, did not conduct or document an independent forward flow test for these check valves.

There has been one previous similar event, in regards to IST program/procedure deficiencies. This was submitted on LER 90-011.

CAUSE:

The cause of this event is procedural deficiency. OST-1131 has never included steps to properly test these two check valves.

SAFETY SIGNIFICANCE:

There were no significant safety consequences as a result of this procedural deficiency. This is based on the following evidence that validates the functional status of these valves and the fact that the systems safety function has never been challenged by radiological conditions. Forward flow is tested through at least one of these check valves once per 18 months by the performance of OST-1231 (Control Room Emergency Filtration Operability Test). This test has consistently verified forward flow and was last performed satisfactorily on 10/1/90. This provides a high degree of confidence that the valves would have functioned as designed if needed.

CORRECTIVE ACTIONS:

1. EPT-172 was specifically developed to perform a forward flow test for check valves ICZ-15 and ICZ-16. This test was successfully completed on 1/16/91 to verify system operability.
2. OST-1131 was revised to incorporate this testing and was performed satisfactorily on 1/18/91.
3. Operations procedure group will review this event to enhance their awareness of potential problems related to ISI program requirements contained in OST procedures.
4. ISI personnel will perform a comprehensive procedure review to identify similar IST program problems, related to the testing of parallel flow through check valves.

EISS CODE INFORMATION:

N/A