



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 6, 2018

Mr. Bryan C. Hanson
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer
Exelon Nuclear
LaSalle County Station
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS TO RENEWED FACILITY OPERATING LICENSES RE: REVISION TO TECHNICAL SPECIFICATIONS 2.1.1 FOR MINIMUM CRITICAL POWER RATIO SAFETY LIMITS (CAC NOS. MG0211 AND MG0212; EPID L-2017-LLA-0279)

Dear Mr. Hanson:

The U.S. Nuclear Regulatory Commission (NRC or Commission) has issued the enclosed Amendment No. 227 to Renewed Facility Operating License No. NPF-11 and Amendment No. 213 to Renewed Facility Operating License No. NPF-18 for the LaSalle County Station (LSCS), Units 1 and 2, respectively. The amendments revise the relevant portions of the technical specifications and license pages in response to your application dated August 29, 2017, as supplemented by letter dated January 25, 2018.

The amendments revised the LSCS, Units 1 and 2, TS 2.1.1, "Reactor Core SLs [Safety Limits]." Specifically, this change incorporates revised LSCS, Units 1 and 2, safety limit for minimum critical power ratio (SLMCPR) for two circulation loop MCPR and single circulation loop MCPR values for Unit 1 and Unit 2 based on the results of the cycle-specific analyses performed by Global Nuclear Fuel (GNF) for LSCS Unit 1, Cycle 17, and LSCS Unit 2, Cycle 17.

B. Hanson

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A copy of the Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "B. Vaidya", with a horizontal line underneath.

Bhalchandra K. Vaidya, Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosures:

1. Amendment No. 227 to NPF-11
2. Amendment No. 213 to NPF-18
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 227
Renewed License No. NPF-11

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated August 29, 2017, as supplemented by letter dated January 25, 2018, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

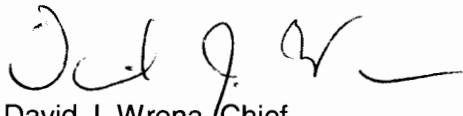
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Renewed Facility Operating License No. NPF-11 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised Through Amendment No. 227, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to startup from the February 2018 refueling outage for Unit 1 (i.e., L1R17) for operation starting in Cycle 18.

FOR THE NUCLEAR REGULATORY COMMISSION



David J. Wrona, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Date of Issuance: February 6, 2018

Attachment:
Revised License and Technical
Specification Pages



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 213
Renewed License No. NPF-18

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated August 29, 2017, as supplemented by letter dated January 25, 2018, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Renewed Facility Operating License No. NPF-18 is hereby amended to read as follows:

- (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 213, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented prior to startup from the February 2018 refueling outage for Unit 1 (i.e., L1R17). This would be a mid-Cycle 17 implementation for Unit 2.

FOR THE NUCLEAR REGULATORY COMMISSION



David J. Wrona, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Date of Issuance: February 6, 2018

Attachment:
Revised License and Technical
Specification Pages

ATTACHMENT TO LICENSE AMENDMENT NOS. 227 AND 213
RENEWED FACILITY OPERATING LICENSE NOS. NPF-11 AND NPF-18
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

Replace the following pages of the Renewed Facility Operating Licenses and Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

License
NPF-11, Page 3
NPF-18, Page 3

TSs
TS 2.0-1

INSERT

License
NPF-11, Page 3
NPF-18, Page 3

TSs
TS 2.0-1

- (3) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- Am. 146
01/12/01 (4) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- Am. 202
07/21/11 (5) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of LaSalle County Station, Units 1 and 2, and such Class B and Class C low-level radioactive waste as may be produced by the operation of Braidwood Station, Units 1 and 2, Byron Station, Units 1 and 2, and Clinton Power Station, Unit 1.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

- Am. 198
09/16/10 (1) Maximum Power Level
The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3546 megawatts thermal).
- Am. 227 (2) Technical Specifications and Environmental Protection Plan
The Technical Specifications contained in Appendix A, as revised through Amendment No. 227, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
- Am. 194
08/28/09 (3) DELETED
- Am. 194
08/28/09 (4) DELETED
- Am. 194
08/28/09 (5) DELETED

- (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- Am. 189
07/21/11 (5) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of LaSalle County Station, Units 1 and 2, and such Class B and Class C low-level radioactive waste as may be produced by the operation of Braidwood Station, Units 1 and 2, Byron Station, Units 1 and 2, and Clinton Power Station, Unit 1.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

Am. 185
09/16/10

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3546 megawatts thermal). Items in Attachment 1 shall be completed as specified. Attachment 1 is hereby incorporated into this license.

Am. 213

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 213, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

2.0 SAFETY LIMITS (SLs)

2.1 SLs

2.1.1 Reactor Core SLs

2.1.1.1 With the reactor steam dome pressure < 700 psia or core flow < 10% rated core flow:

THERMAL POWER shall be \leq 25% RTP.

2.1.1.2 With the reactor steam dome pressure \geq 700 psia and core flow \geq 10% rated core flow:

For Unit 1, MCPR shall be \geq 1.11 for two recirculation loop operation or \geq 1.13 for single recirculation loop operation.

For Unit 2, MCPR shall be \geq 1.12 for two recirculation loop operation or \geq 1.15 for single recirculation loop operation.

2.1.1.3 Reactor vessel water level shall be greater than the top of active irradiated fuel.

2.1.2 Reactor Coolant System Pressure SL

Reactor steam dome pressure shall be \leq 1325 psig.

2.2 SL Violations

With any SL violation, the following actions shall be completed within 2 hours:

2.2.1 Restore compliance with all SLs; and

2.2.2 Insert all insertable control rods.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 227 TO RENEWED FACILITY OPERATING
LICENSE NO. NPF-11 AND AMENDMENT NO. 213 TO RENEWED FACILITY
OPERATING LICENSE NO. NPF-18
EXELON GENERATION COMPANY, LLC
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374
CAC NOS. MG0211 AND MG0212; EPID NO. L-2017-LLA-0279

1.0 INTRODUCTION

By letter dated August 29, 2017 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17241A283), as supplemented by letter dated January 25, 2018 (ADAMS Accession No. ML18025B874), Exelon Generation Company, LLC (EGC or the licensee), requested amendments to the Renewed Facility Operating License Nos. NPF-11 and NPF-18 for the LaSalle County Station (LSCS), Units 1 and 2, to revise the LSCS technical specifications (TSs).

The proposed change would revise Technical Specification (TS) 2.1.1, "Reactor Core SLs [Safety Limits]," for LSCS, Units 1 and 2. Specifically, the proposed change incorporates revised Safety Limit Minimum Critical Power Ratios (SLMCPRs) due to the cycle specific analyses performed by Global Nuclear Fuel (GNF) for LSCS, Unit 1, Cycle 17, and LSCS, Unit 2, Cycle 17.

The supplemental submission dated January 25, 2018, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC or the Commission) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on December 5, 2017 (82 FR 57482).

2.0 REGULATORY EVALUATION

2.1 Proposed Changes

On the basis of the calculations for LSCS, Units 1 and 2, core reload analysis for Operating Cycle 17, the calculated SLMCPR would change for Unit 1, from ≥ 1.13 to ≥ 1.11 for two recirculation loop operation (TLO) and from ≥ 1.15 to ≥ 1.13 for single recirculation loop operation (SLO); and for Unit 2, from ≥ 1.14 to ≥ 1.12 for TLO and from ≥ 1.17 to ≥ 1.15 for SLO. Accordingly, the licensee proposes to revise LSCS TS Section 2.1.1.2 to read as follows:

With the reactor steam dome pressure ≥ 700 psia and core flow $\geq 10\%$ rated core flow:

For Unit 1, MCPR [minimum critical power ratio] shall be ≥ 1.11 for two recirculation loop operation or ≥ 1.13 for single recirculation loop operation.

For Unit 2, MCPR shall be ≥ 1.12 for two recirculation loop operation or ≥ 1.15 for single recirculation loop operation.

Implementation Plan for the Proposed Amendments

In the supplemental submission dated January 25, 2018, the licensee stated that:

Unit 1: Once approved, the Unit 1 amendment shall be implemented prior to startup from the February 2018 refueling outage for Unit 1 (i.e., L1R17) for operation starting in Cycle 18.

Unit 2: Once approved, the Unit 2 amendment shall be implemented prior to startup from the February 2018 refueling outage for Unit 1 (i.e., L1R17). This will be a mid-Cycle 17 implementation for Unit 2.

2.2 Regulations and Guidance

The regulatory requirements and guidance documents that the NRC staff considered in its review of the proposed amendment included the following:

- Section 182a of the Atomic Energy Act requires applicants for nuclear power plant operating licenses to include TSs as part of the license. The TSs ensure the operational capability of structures, systems, and components that are required to protect the health and safety of the public. The NRC's regulatory requirements related to the content of the TSs are contained in Section 50.36, "Technical specifications," of Title 10 of the *Code of Federal Regulations* (10 CFR), which requires that the TSs include items in the following specific categories: (1) safety limits (SLs), limiting safety systems settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in TSs.
- The fuel cladding is one of the physical barriers that separate the radioactive materials from the environment. The SLMCPR is an SL that is required to be in TS to ensure that fuel design limits are not exceeded. The SLMCPR limit is contained in LSCS TS 2.1.1.2. The regulations in 10 CFR 50.36(c)(1)(i)(A) state, in part, that safety limits for nuclear reactors are limits upon important process

variables that are found to be necessary to reasonably protect the integrity of certain of the physical barriers that guard against the uncontrolled release of radioactivity.

- General Design Criterion (GDC) 10, "Reactor Design," of Appendix A to 10 CFR Part 50 states that the reactor core and associated coolant, control, and protection systems shall be designed with appropriate margin to assure that Specified Acceptable Fuel Design Limits (SAFDLs) are not exceeded during any condition of normal operation, including the effects of anticipated operational occurrences.
- Guidance on the acceptability of the reactivity control systems, the reactor core, and fuel system design is provided in NUREG-0800, "Standard Review Plan (SRP) for the Review of Safety Analysis Reports for Nuclear Power Plants." Specifically, SRP Section 4.2, "Fuel System Design" (ADAMS Accession No. ML070740002), specifies the fuel damage criteria for evaluation of whether fuel designs meet the SAFDLs. SRP Section 4.4, "Thermal and Hydraulic Design" (ADAMS Accession No. ML070550060) provides guidance on the review of thermal-hydraulic design in meeting the requirement of GDC 10 and the fuel design criteria established in SRP Section 4.2. It states that the critical power ratio (CPR) is to be established such that at least 99.9 percent of fuel rods in the core would not be expected to experience departure from nucleate boiling or boiling transition during normal operation or anticipated operational occurrences.

3.0 TECHNICAL EVALUATION

The SLMCPR numeric values in LSCS TS 2.1.1.2 are SLs. The SLMCPR limit is established such that at least 99.9 percent of the fuel rods in the core would not be expected to experience the onset of transition boiling as a result of normal operation and transients, which in turn ensures fuel cladding damage does not occur. The SLMCPR limit is established such that fuel design limits are not exceeded during steady state operation, normal operational transients, and abnormal operational transients. As such, fuel damage is calculated not to occur if the limit is not violated. However, because fuel damage is not directly observable, a step-back approach is used to establish corresponding operating limits. The operating limit MCPR (OLMCPR) is established by summing the cycle-specific core reload transient analyses adders and the calculated SLMCPR values. The OLMCPR is required to be established and documented in the core operating limits report (COLR) for each reload cycle by LSCS TS 5.6.5, COLR.

The absolute value of SLMCPR tends to vary cycle-to-cycle, typically due to the introduction of improved fuel bundle types, changes in fuel vendors or applicable computer codes, and changes in core loading pattern. Following the determination of the cycle-specific SLMCPR values, the OLMCPR values are derived. The cycle-specific SLMCPR numeric values are listed in LSCS TS 2.1.1.2, and, therefore, must be revised using the license amendment process.

Global Nuclear Fuel (GNF) performed the LSCS Units 1 and 2 Cycle 17 SLMCPR calculation consistent with NRC-approved methodologies and uncertainties, as documented in the following topical reports (TRs):

- NEDE-24011-P-A "General Electric Standard Application for Reactor Fuel," Revision 22, November 2015 (GESTAR II) (ADAMS Accession Nos. ML15324A145, ML15324A148 and ML15324A149)

- NEDC-32601P-A, "Methodology and Uncertainties for Safety Limit MCPR Evaluations," August 1999 (ADAMS Accession No. ML14093A216)
- NEDC-32694P-A, "Power Distribution Uncertainties for Safety Limit MCPR Evaluations," August 1999 (ADAMS Accession No. ML993140059)
- NEDC-32505P-A, "R-Factor Calculation Method for GE 11, GE 12 and GE 13 Fuel," Revision 1, July 1999 (ADAMS Accession No. ML060520636)

These methodologies were used for the LSCS Cycles 16 and 17 SLMCPR calculations for both units. The NRC staff reviewed the proposed change to ensure that the generic methods were appropriately applied to LSCS. The LSCS Unit 1 Cycle 17 core will be a full core of GNF2 fuel assemblies, and Unit 2 Cycle 17 core will be a full core of GNF2 fuel assemblies and four GNF3 Lead Use Assemblies (LUAs). No plant hardware or operational changes are required with this proposed change.

NEDC-32505P-A is the generic R-Factor methodology report that describes the changed methodology that was adopted after part length rods were introduced. The NRC staff's safety evaluation for NEDC-32505P-A has a requirement that the applicability of the R-Factor methodology be confirmed when a new fuel type is introduced. The GNF letter designated FLN-2007-011, "GNF2 Advantage Generic Compliance with NEDE-24011-P-A (GESTAR II), NEDC-33270P, March 2007, and GEXL17 Correlation for GNF2 Fuel, NEDC-33292P, March 2007," was submitted to the NRC on March 14, 2007 (ADAMS Accession No. ML070780335). FLN-2007-011 confirmed that the R-factor methodology of NEDC-32505P-A is applicable to GNF2, and that all of the criteria defined in NEDE-24011-P-A have been met for the GNF2 fuel design. As part of an NRC audit related to this report, the GNF2 fuel design was verified to have been evaluated in accordance with the TRs listed above. This was documented in an audit report dated September 25, 2008 (ADAMS Accession No. ML081630579).

In Page 6, Attachment 4, of the application dated August 29, 2017, the licensee stated that LaSalle Unit 2 Cycle 17 includes GNF3 LUAs, and that these bundles were located in the core such that the SLMCPR calculation predicts that they have no rods in transition boiling when the core is at the SLMCPR.

In LSCS TS 4.2.1, "Fuel Assemblies," it is stated, in part,

A limited number of lead test assemblies that have not completed representative testing may be placed in non-limiting core regions.

Consistent with LSCS TS 4.2.1, the licensee placed a limited number (four) of LUAs in non-limiting core regions in Unit 2, which is, therefore, acceptable.

On the basis of the analysis performed by GNF using the NRC-approved methodologies described above, the licensee has proposed to amend the LSCS, Units 1 and 2, TS Section 2.1.1.2, to revise the SLMCPR for Cycle 17. This information regarding requested changes to the LSCS, Units 1 and 2 TS SLMCPR is based on and is for the core rated power of 3,546 MWt (megawatt), and at minimum core flow of 82.8 percent at rated power.

The current required SLMCPR values in LSCS TS is ≥ 1.13 for TLO and ≥ 1.15 for SLO for Unit 1, and ≥ 1.14 for TLO and ≥ 1.17 for SLO for Unit 2. Calculations performed by GNF for Cycle 17 resulted in a minimum calculated value of SLMCPR to be ≥ 1.11 for TLO and ≥ 1.13 for

SLO for Unit 1, and ≥ 1.12 for TLO and ≥ 1.15 for SLO for Unit 2. For Cycle 17, the minimum core flow SLMCPR calculation performed at 82.8 percent core flow and rated core power condition was limiting as compared to the rated core flow and rated core power condition. GNF's calculation of the revised plant-specific SLMCPR numeric values for LSCS Cycle 17 was performed as part of the reload licensing analysis for LSCS Cycle 17, and is based upon NRC-approved methods, therefore it is acceptable.

The NRC staff verified that no departures from NRC-approved methodologies, or deviations from NRC-approved calculational uncertainties, were identified in the LSCS, Cycle 17, SLMCPR calculations. All calculated uncertainties for LSCS, Cycle 17, SLMCPR calculations were conservative relative to NRC-approved values.

The NRC staff further verified that the proposed changes would continue to meet the applicable regulations and requirements, and that the analysis performed to calculate the LSCS Cycle 17 SLMCPR numeric values was based upon NRC-approved methodologies. The NRC staff concludes that the SLMCPR will continue to provide assurance that 99.9 percent of the fuel rods in the core will not exceed the CPR, and that fuel cladding integrity will be maintained under conditions of normal operation and with appropriate margin for anticipated operational occurrences.

Technical Evaluation Conclusion

The NRC staff finds that the licensee's proposed amendment to update the TSs to include cycle-specific SLMCPR numeric values is based on NRC-approved methodologies that have been approved for use with GNF2 fuel. The amendment is consistent with the regulatory requirements and guidance as discussed in Section 2.0 of this safety evaluation, and, therefore, is acceptable. The NRC staff determined that the changes do not require any exemptions or relief from regulatory requirements. Defense-in-depth and sufficient safety margins will continue to be maintained. Therefore, based on the above considerations, the NRC staff finds that the proposed changes to revise the SLMCPR values in TS 2.1.1.2 from ≥ 1.13 to ≥ 1.11 for TLO and from ≥ 1.15 to ≥ 1.13 for SLO for Unit 1, and from ≥ 1.14 to ≥ 1.12 for TLO and from ≥ 1.17 to ≥ 1.15 for SLO for Unit 2, as well as the licensee's plan for implementation of these amendments for Unit 1 and Unit 2, are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment on January 9, 2018. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (December 5, 2017 (82 FR 57482)). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no

environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: M. Razzaque

Date of Issuance: February 6, 2018

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2 – ISSUANCE OF AMENDMENTS TO RENEWED FACILITY OPERATING LICENSES RE: REVISION TO TECHNICAL SPECIFICATIONS 2.1.1 FOR MINIMUM CRITICAL POWER RATIO SAFETY LIMITS (CAC NOS. MG0211 AND MG0212; EPID L-2017-LLA-0279) DATED FEBRUARY 6, 2018

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- M. Razzaque, NRR/SRXB
- M. Panicker, NRR/SNPB

ADAMS Accession No.: ML18008A123

(*) by e-mail or memo

OFFICE	NRR/DORL/LPL3/PM	NRR/DORL/LPL3/LA	NRR/DSS/STSB/BC	NRR/DSS/SRXB/BC
NAME	BVaidya	SRohrer	VCusumano	JWhitman
DATE	1/8/2018	1/8/2018	1/11/2018	12/20/2018
OFFICE	NRR/DSS/SNPB/BC	OGC – NLO	NRR/DORL/LPL3/BC	NRR/DORL/LPL3/PM
NAME	Lukes	STurk	DWrona	BVaidya
DATE	1/11/2018	1/26/2018	2/5/2018	2/6/2018

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