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SUBJECT: Suppl to util 890417 application for amend to License NPF-63,revising TS 6.9.1.6 re core operating limits rept.

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A. B CUTTER
Vice President
Nuclear Services Department

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT
DOCKET NO. 50-400/LICENSE NO. NPF-63
SUPPLEMENT TO CYCLE 3 RELOAD AMENDMENT REQUEST

Gentlemen:

Carolina Power & Light Company (CP&L) hereby submits a supplement to the April 17, 1989 license amendment request concerning Technical Specification (TS) changes in support of the Cycle 3 reload for the Shearon Harris Nuclear Power Plant. This letter revises TS 6.9.1.6 concerning the Core Operating Limits Report. These changes are administrative in nature and are made in response to NRC comments.

The revised TS page is attached. This change is administrative in nature and as such the 10CFR50.92 Evaluation and the Environmental Evaluation provided in the Company's April 17, 1989 submittal remain valid.

Please refer any questions regarding this submittal to Mr. John Eads at (919) 546-4165.

Yours very truly,

A. B. Cutter

ABC/JHE/crs (374CRS)

Attachment

cc: Mr. R. A. Becker
Mr. W. H. Bradford
Mr. Dayne H. Brown
Mr. S. D. Ebneter



A001
1/1

PEAKING FACTOR LIMIT REPORT ← REPLACE WITH ATTACHED INSERT

6.9.1.6 The W(Z) Functions for RAOC and Base Load operation and the value for $APL^{MD}$ (as required) shall be established for each reload core and implemented prior to use.

The methodology used to generate the W(Z) functions for RAOC and Base Load operation and the value for $APL^{MD}$ shall be those previously reviewed and approved by the NRC.* If changes to these methods are deemed necessary, they will be evaluated in accordance with 10 CFR 50.59 and submitted to the NRC for review and approval prior to their use if the change is determined to involve an unreviewed safety question or if such a change would require amendment of previously submitted documentation.

A report containing the W(Z) function for RAOC and Base Load operation and the value for $APL^{MD}$ (as required) shall be provided to the NRC in accordance with 10 CFR 50.4 within 30 days after each cycle initial criticality.

Any information needed to support W(Z), $W(Z)_{BL}$, and $APL^{MD}$ will be by request from the NRC and need not be included in this report.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Regional Administrator of the Regional Office of the NRC within the time period specified for each report.

6.10 RECORD RETENTION

6.10.1 In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.2 The following records shall be retained for at least 5 years:

a. Records and logs of unit operation covering time interval at each power level;

b. Records and logs of principal maintenance activities, inspections, repair, and replacement of principal items of equipment related to nuclear safety;

c. All REPORTABLE EVENTS;

d. Records of surveillance activities, inspections, and calibrations required by these Technical Specifications;



* WCAP-10216, "Relaxation of Constant Axial Offset Control-$F_Q$ Surveillance Technical Specification."

INSERT TO TS PAGE 6-24

6.9.1.6 CORE OPERATING LIMITS REPORT

6.9.1.6.1 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT prior to each reload cycle, or prior to any remaining portion of a reload cycle, for the following:

a. The shutdown rod insertion limits of Specification 3.1.3.5.

b. The control rod insertion limits of Specification 3.1.3.6.

c. The axial flux difference of Specification 3.2.1.

d. The surveillance requirements of Specifications 4.2.2.2, 4.2.2.3 and 4.2.2.4.

6.9.1.6.2 The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:

a. WCAP-10216-P-A, Relaxation of Constant Axial Offset Control $F_Q$ Surveillance Technical Specification, 1983.

b. WCAP-9272-P-A, Westinghouse Reload Safety Evaluation Methodology, 1985.

6.9.1.6.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements, shall be provided, upon issuance for each reload cycle, to the NRC Document Control Desk, with copies to the Regional Administrator and Resident Inspector.