

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

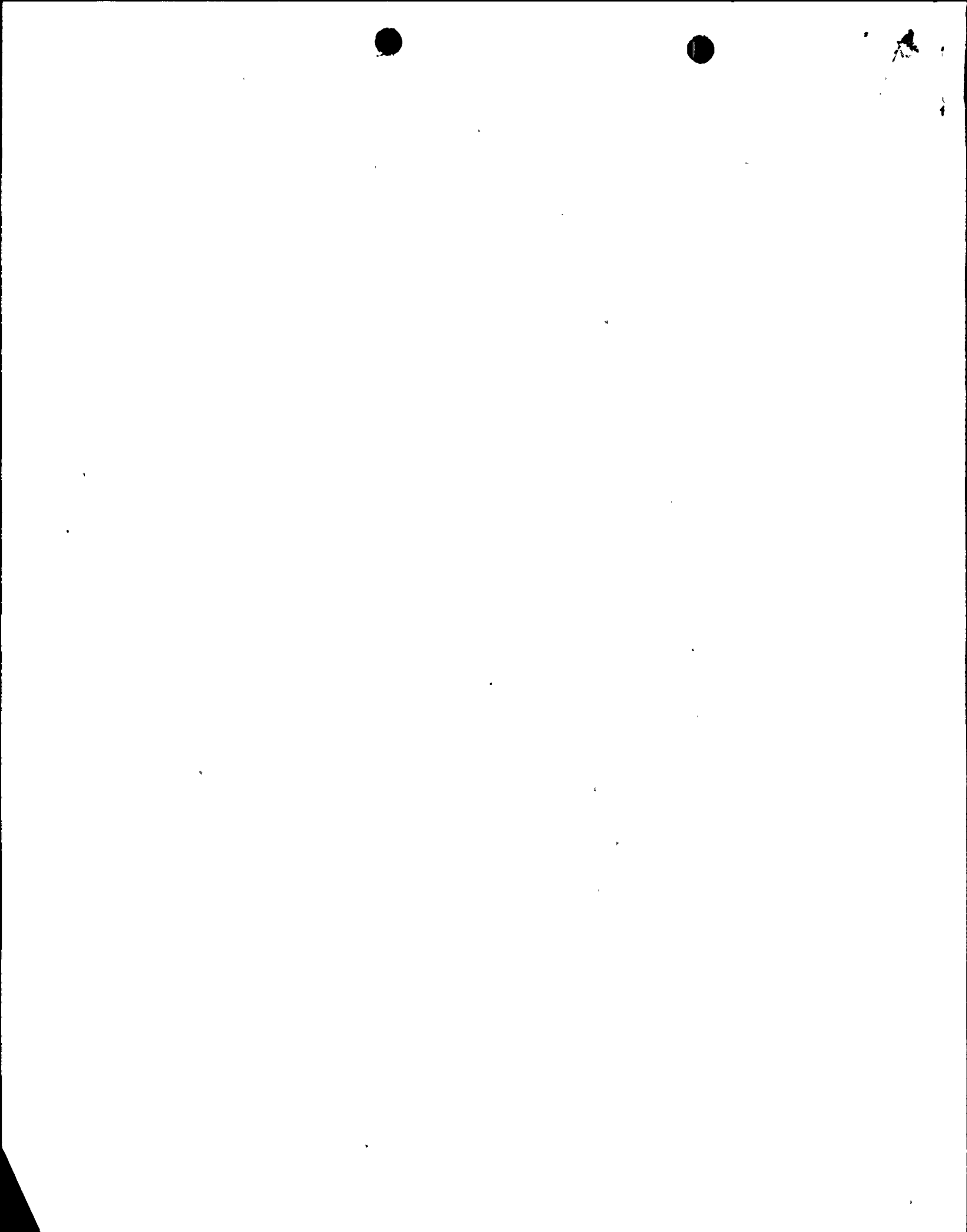
ACCESSION NBR: 8705280348 DOC. DATE: 87/05/20 NOTARIZED: NO DOCKET #
 FACIL: 50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina 05000400
 AUTH. NAME AUTHOR AFFILIATION
 SCHWABENBAUER Carolina Power & Light Co.
 WATSON, R. A. Carolina Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-023-00: on 870420, alternate miniflow valves for charging safety injection pump found isolated. Verification of correct valve lineup unavailable. Cause unknown. Procedure OP-107 will be revised. W/870520 1tr.

DISTRIBUTION CODE: IE22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: Application for permit renewal filed. 05000400

	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTTR ENCL
	PD2-1 LA	1 1	PD2-1 PD	1 1
	BUCKLEY, B	1 1		
INTERNAL:	ACRS MICHELSON	1 1	ACRS MOELLER	2 2
	AEOD/DOA	1 1	AEOD/DSP/ROAB	2 2
	AEOD/DSP/TPAB	1 1	DEDRO	1 1
	NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
	NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
	NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
	NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
	NRR/DEST/RSB	1 1	NRR/DEST/SGB	1 1
	NRR/DLPQ/HFB	1 1	NRR/DLPQ/QAB	1 1
	NRR/DOEA/EAB	1 1	NRR/DREP/RAB	1 1
	NRR/DREP/RPB	2 2	NRR/PMAS/ILRB	1 1
	NRR/PMAS/PTSB	1 1	<u>REG FILE</u> 02	1 1
	RES DEPY GI	1 1	RGN2 FILE 01	1 1
EXTERNAL:	EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
	LPDR	1 1	NRC PDR	1 1
	NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1



LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SHEARON HARRIS PLANT UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 4 0 0	PAGE (3) 1 OF 0 3
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TITLE (4)
ISOLATED MINIFLOW PATH FOR CSIP 1C-SAB

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	4	2 0 8 7	8	7	0 2 3	0	5	2 0 8 7			0 5 0 0 0

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 9 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)						
	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)						
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)

NAME RICHARD SCHWABENBAUER - REGULATORY COMPLIANCE	TELEPHONE NUMBER
	AREA CODE 9 1 9
	3 6 2 - 2 6 6 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On April 20, 1987 the plant was operating at 90 percent reactor power. At 0900 hours the alternate miniflow valves for 1C-SAB Charging Safety Injection Pump (CSIP) were found to be isolated. This flow path is required to be operable in accordance with FSAR table 6.3.2-8. 1C-SAB CSIP was placed in service on February 21, 1987 in place of 1B-SB CSIP. 1C-SAB CSIP was placed in service, at the time, in accordance with OP-107, Chemical and Volume Control System, which gives the required valve line up. However, it cannot be verified if the lineup was performed or if the valves were subsequently mispositioned.

Upon discovery of the misaligned valves, a new valve lineup was immediately initiated and was successfully completed at 1015 hours in accordance with OP-107 and PLP-702, "Independent Verification". OP-107 is being revised to include attachments which will be completed, signed, and verified when placing a CSIP in service to ensure valve line up is correct.

8705280348 870520
PDR ADDOCK 05000400
S PDR

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Shearon Harris Plant Unit 1	DOCKET NUMBER (2) 050004087	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		87	023	010	02	OF 013

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION:

On April 20, 1987 the plant was operating at 90 percent reactor power. At 0900 hours the alternate miniflow valves for 1C-SAB Charging Safety Injection Pump (CSIP) were found to be isolated. The as found positions of the valves were 1CS-749-shut, 1CS-750-partially open, and 1CS-751-open. The correct valve positions are 1CS-749-open, 1CS-750-open, and 1CS-751-closed, when 1C-SAB CSIP is in use in place of 1B-SB CSIP. The flow path is required to be operable in accordance with FSAR table 6.3.2-8. 1C-SAB CSIP was declared operable on February 21, 1987 in place of 1B-SB CSIP.

The incorrect lineup isolated the pump discharge from the miniflow path to the RWST. (The 1A-SA pump was not affected by this lineup error.)

1C-SAB CSIP was placed in service in accordance with OP-107, Chemical and Volume Control System, Section 8.17, which states the required valve positioning. However, a PLP-702, Independent Verification, form cannot be located to verify the proper valve positioning at the time 1C-SAB CSIP was placed in service. OP-107 does not contain a signoff list for this evolution; the forms and instructions of PLP-702 are used to obtain this documentation.

Upon discovery of the mispositioned valves, a new valve lineup was initiated and successfully completed at 1015 hours in accordance with OP-107 and PLP-702.

CAUSE:

The event was caused by personnel error. Either the valves were not properly positioned when the pump was placed into service or were mispositioned later by persons unknown. Procedural requirements were not met when the 1C-SAB CSIP was placed in service in that the valve lineup verification sheet was not completed and filed or was later lost.

ANALYSIS:

1A-SA CSIP was operable and properly aligned throughout the entire period. The 1C-SAB CSIP was capable of normal operation and capable of starting on a Safety Injection Signal throughout the time frame the alternate miniflow valves were improperly positioned.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Shearon Harris Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 0 0	LER NUMBER (6)			PAGE (3)	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

ANALYSIS (continued)

The motor operated alternate miniflow isolation valves open on a SI Signal and place two relief valves in service (refer to FSAR Figure 9.3.4-2 and -3). The relief valves lift at 2500 psia to provide recirculation flow for the CSIP. The alternate miniflow system provides protection only during certain steam line or feedwater line breaks. It was determined by Westinghouse that for these breaks, the potential exists to have the RCS pressure increase to the point where the CSIP would be dead headed prior to Safety Injection termination. Prior to SI termination, the CSIP's normal miniflow path would be isolated and the alternate miniflow system was designed to prevent pump damage during this scenario. The design basis for the alternate miniflow assumes that the Pressurizer PORV's do not function because they are not safety related.

CORRECTIVE ACTION:

Procedure OP-107 is being revised to include a valve line-up for the alternate mini-flow which will have to be completed, signed, and verified when placing a CSIP into service.



Carolina Power & Light Company

HARRIS NUCLEAR PROJECT
P.O. Box 165
New Hill, NC 27562
MAY 20 1987

File Number: SHF/10-13510C
Letter Number: HO-870433 (0)

U.S. Nuclear Regulatory Commission
ATTN: NRC Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT UNIT 1
DOCKET NO. 50-400
LICENSE NO. NPF-63
LICENSEE EVENT REPORT 87-023-00

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September, 1983.

Very truly yours,

R. A. Watson
Vice President
Harris Nuclear Project

RAW:bjb

Enclosure

cc: Dr. J. Nelson Grace (NRC - RII)
Mr. B. Buckley (NRR)
Mr. G. Maxwell (NRC - SHNPP)

