

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 8704160109 DOC. DATE: 87/04/10 NOTARIZED: NO DOCKET #
 FACIL: 50-400 Shearon Harris Nuclear Power Plant, Unit 1, Carolina 05000400
 AUTH. NAME AUTHOR AFFILIATION
 HUDSON, D. N. Carolina Power & Light Co.
 WATSON, R. A. Carolina Power & Light Co.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 87-014-00: on 870314, three pressure guages installed.
 placed in svc & opened on three drain lines whose isolation
 valves considered containment isolation valves. Caused by
 personnel error. Gauges removed. W/870410 ltr.

DISTRIBUTION CODE: 1E22D COPIES RECEIVED: LTR 1 ENCL 1 SIZE: 4
 TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

NOTES: Application for permit renewal filed. 05000400

RECIPIENT ID CODE/NAME	COPIES LTR ENCL	RECIPIENT ID CODE/NAME	COPIES LTR ENCL
PD2-1 LA	1 1	PD2-1 PD	1 1
BUCKLEY, B	1 1		

INTERNAL: ACRS MICHELSON	1 1	ACRS MOELLER	1 1
ACRS WYLIE	1 1	AEOD/DOA	1 1
AEOD/DSP/ROAB	2 2	AFOD/DSP/TAPB	1 1
AEOD/DSP/TPAB	1 1	NRR/ADT	1 1
NRR/DEST/ADE	1 0	NRR/DEST/ADS	1 0
NRR/DEST/CEB	1 1	NRR/DEST/ELB	1 1
NRR/DEST/ICSB	1 1	NRR/DEST/MEB	1 1
NRR/DEST/MTB	1 1	NRR/DEST/PSB	1 1
NRR/DEST/RSB	1 1	NRR/DEST/SGB	1 1
NRR/DLPQ/HFB	1 1	NRR/DLPQ/QAB	1 1
NRR/DOEA/EAB	1 1	NRR/DREP/EPB	1 1
NRR/DREP/RAB	1 1	NRR/DREP/RPB	2 2
NRR/PMAS/ILRB	1 1	NRR/PMAS/PTSB	1 1
REG FILE 02	1 1	RES SPEIS, T	1 1
RGN2 FILE 01	1 1		

EXTERNAL: EG&G GROH, M	5 5	H ST LOBBY WARD	1 1
LPDR	1 1	NRC PDR	1 1
NSIC HARRIS, J	1 1	NSIC MAYS, G	1 1

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) SHEARON HARRIS PLANT, UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 4 0 0	PAGE (3) 1 OF 03
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TITLE (4)
CONTAINMENT INTEGRITY

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
03	14	87	87	014	00	04	10	87		0 5 0 0 0	

OPERATING MODE (9) 4	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)									
POWER LEVEL (10)	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)						
	20.405(a)(1)(i)	50.38(c)(1)	50.73(a)(2)(v)	73.71(c)						
	20.405(a)(1)(ii)	50.38(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)							
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)							
20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)									
NAME O. N. HUDSON, SENIOR ENGINEER, REGULATORY COMPLIANCE							TELEPHONE NUMBER AREA CODE 919 362-2363		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)					EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO								

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract

Three test pressure gauges were installed and placed in service on each of three drain lines whose isolation valves are considered containment isolation valves. The isolation valves were then opened to place the gauges in service. This contradicts the Technical Specification requirement that these valves be maintained closed for containment integrity. Upon discovery, the valves were closed, the test gauges removed and the pipe recapped. A pipe hanger on one drain line was found to be loose, and was subsequently retightened.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
SHEARON HARRIS PLANT, UNIT 1	0500040087	—	014	—	0	02	OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Description of Event

The plant was operating in Mode 1 at approximately 50% power when three temporary pressure gauges were installed on drain lines of the Auxiliary Feedwater System. Each drain line is located downstream of the isolation valve for each of the three steam generators, inside the steam tunnel which is outside of the Reactor Containment Building. The drain lines were to be used as taps to monitor the pressure of main feedwater during evaluations of Auxiliary Feedwater check valve backleakage. Removal of the drain pipe cap was required to install each gauge and opening of the two isolation valves was required to place each gauge in service.

The first gauge was installed and placed in service March 12, 1987. The second and third gauges were installed and placed in service March 13, 1987. The reactor tripped and was shut down to Mode 4 later on March 13. OST-1029, "Containment Penetration Outside Isolation Valve Verification," was conducted on March 14, 1987 and identified the open isolation valves which are required to be closed per Technical Specification 3.6.1.1, Containment Integrity. A loosened hanger "U-Bolt" supporting the drain line associated with the A steam generator was also reported at this time. The time of discovery was 0655. As a result, the outside drain valves were closed, gauges removed and pipe caps installed by 0730 the same day. Maintenance was requested to tighten the loosened hanger. No components were declared inoperable since the plant was in Mode 4 and Auxiliary Feedwater is not required to be operable.

Cause

This event is being reported in accordance with 10CFR50.73(a)(2)(i)(B) due to containment isolation valves being open for longer than allowed per the Technical Specifications.

The gauges were installed due to personnel error in failing to recognize the drain valves as containment isolation valves.

Analysis

No adverse effects occurred as a result of these valves being opened. No equipment or systems failed as a result of these valves being opened. Other than cognitive error of personnel involved, no actions by operators or other personnel affected this event. No safety system responses occurred as a result of this event.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
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SHEARON HARRIS PLANT, UNIT 1	0 5 0 0 0 4 0 0	8 7	— 0 1 4	— 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Analysis (continued)

The safety consequences and implications are minimal. The drain lines are on secondary plant equipment and therefore the probability of radiological release is small. Although the gauges installed were not safety related two had ranges of 0-3000 psi, and the third 0-5000 psi. This exceeds considerably the Auxiliary Feedwater System design pressure of 1185 psig and containment accident pressures of up to 41 psig that could be experienced during an accident.

No previous similar events have occurred.

Corrective Actions

Corrective actions taken were the removal of gauges, the closure of drain isolation valves and reinstallation of the pipe cap of all three drain lines to restore containment integrity. A work request was written March 14, 1987 to tighten the loose hanger. Other actions being evaluated are: (1) to identify all other containment isolation valves of this nature and physically mark them in a manner to identify them as such to preclude future occurrences of similar events, and (2) to include in the Equipment Data Base System, used to control maintenance work, a note identifying all such valves as part of the containment isolation boundary.



Carolina Power & Light Company

HARRIS NUCLEAR PROJECT
P.O. Box 165
New Hill, NC 27562
APR 10 1987

File Number: SHF/10-13510C
Letter Number: HO-870403 (O)

U.S. Nuclear Regulatory Commission
ATTN: NRC Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT UNIT 1
DOCKET NO. 50-400
LICENSE NO. NPF-63
LICENSEE EVENT REPORT 87-014-00

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September, 1983.

Very truly yours,

R. A. Watson
Vice President
Harris Nuclear Project

RAW:skm

Enclosure

cc: Dr. J. Nelson Grace (NRC - RII)
Mr. B. Buckley (NRC)
Mr. G. Maxwell (NRC - SHNPP)

