

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 76
(1 point)

Given the following:

- The plant is operating at 100% RTP
- A planned maintenance item is scheduled for the shift has a YELLOW risk status

Subsequently:

- PCV-455C, PZR PORV fully opens
- Attempts to close the valve have failed
- RC-536, PORV BLOCK Valve is manually CLOSED
- The plant remains on line

Which ONE of the following correctly completes the statements below?

Based on these conditions, Pressurizer PORV Block Valve RC-536 is (1) .

- The updated Risk Profile for the planned maintenance reveals that the risk is an "Elevated YELLOW" status.

IAW OMM-048, WORK COORDINATION AND RISK ASSESSMENT, the planned maintenance item (2) be released for work this shift without authorization from the Plant Manager, due to the change in risk status.

- A. (1) OPERABLE
(2) can
 - B. (1) inoperable
(2) can
 - C. (1) OPERABLE
(2) can NOT
 - D. (1) inoperable
(2) can NOT
-

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Question: 77
(1 point)

Given the following:

- The plant tripped from 100% power
- RCS Bleed and Feed has been established per FRP-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK

Current Conditions:

- AFW flow has been restored
- All Narrow Range S/G levels remain OFF-SCALE LOW

Which ONE of the following correctly completes the statements below?

Operators (1) required to establish a set Narrow Range level in at least one S/G before terminating RCS Bleed and Feed.

Operators will terminate RCS Bleed and Feed in (2) .

- A. (1) are
(2) FRP-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK
 - B. (1) are
(2) EOP-E-1, LOSS OF REACTOR OR SECONDARY COOLANT
 - C. (1) are NOT
(2) FRP-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK
 - D. (1) are NOT
(2) EOP-E-1, LOSS OF REACTOR OR SECONDARY COOLANT
-

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Question: 78
(1 point)

Given the following:

- A Station Blackout has occurred from 100% power
- The crew is performing actions of ECA-0.0, LOSS OF ALL AC POWER
- Immediate actions are complete
- Offsite power is available to re-energize the Start-up Transformer (SUT)
- APP-009-B5, MAIN TRANSF FAULT TRIP, alarm is ILLUMINATED.

Subsequently:

- E-1 is restored with EDG "A"
- Transmission Maintenance has taken action such that APP-009-B5 is now extinguished.

Which ONE of the following correctly completes the statements below?

IAW EOP-ECA-0.0, the CRS is required to use procedure _____ (1) _____ to restore power via the SUT.

Prior to restoring power via the SUT, the appropriate procedure _____ (2) _____ require the crew to ensure the Main Generator Lockout Relays 86P and 86BU are reset.

OP-603, ELECTRICAL DISTRIBUTION
OP-603-3, RESETTING HIGH IMPEDANCE FAULTS
86P, GENERATOR LOCKOUT RELAY, PRIMARY
86BU, GENERATOR LOCKOUT RELAY, BACKUP

- A. (1) OP-603
(2) does NOT
 - B. (1) OP-603
(2) does
 - C. (1) OP-603-3
(2) does NOT
 - D. (1) OP-603-3
(2) does
-

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Question: 79
(1 point)

Given the following:

- The plant is in MODE 3 following a reactor trip
- No Battery Chargers are operating
- APP-036-D3, BATT A/B LO VOLT, has alarmed
 - Battery A voltage is 122.8 Volts and slowly lowering
 - Battery B voltage is 125.0 Volts and slowly lowering

Subsequently, it is determined that Battery voltages are lowering at the following rates:

- Battery A - 0.8 volts/minute
- Battery B - 0.5 volts/minute

Which ONE of the following correctly completes the statements below?

Assuming the Battery voltages lower at a constant rate, the approximate time frame that voltages will lower to the required EAL thresholds is ____ (1) ____ minutes.

- At 1000, battery voltages lowered to the required EAL thresholds
- At 1002, maintenance reports that restoration of one Battery Charger will take at LEAST 30 minutes
- At 1005, the Shift Manager declared a Site Area Emergency SS2.1

The declaration at 1005 ____ (2) ____ contrary to the Robinson Nuclear Plant Emergency Plan.

REFERENCE PROVIDED

- A. (1) 30-32 minutes
(2) was
 - B. (1) 30-32 minutes
(2) was NOT
 - C. (1) 36-38 minutes
(2) was NOT
 - D. (1) 36-38 minutes
(2) was
-

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Question: 80
(1 point)

Given the following:

- The plant is operating at 100% power
- Emergency Diesel Generator (EDG) "B" is running paralleled to Bus E-2
- The Control Room has just received a notification of an imminent cyber-attack on the grid from the NRC
- AOP-026, GRID INSTABILITY, has been implemented

Which ONE of the following correctly completes the statements below?

IAW AOP-026, operators are required to _____ (1) _____.

EPP-25, ENERGIZING SUPPLEMENTAL PLANT EQUIPMENT USING THE DSDG, _____ (2) _____ required to be implemented for these conditions.

- A. (1) shutdown EDG "B"
(2) is
 - B. (1) shutdown EDG "B"
(2) is NOT
 - C. (1) ensure EDG "B" is operating as the sole source of power to Bus E-2
(2) is
 - D. (1) ensure EDG "B" is operating as the sole source of power to Bus E-2
(2) is NOT
-

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Question: 81
(1 point)

Given the following:

- The crew is implementing FRP-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK
- The CST is intact, level is 15% and lowering

Which ONE of the following correctly completes the statements below?

The order in which the CRS is required to attempt to restore Feed Flow to S/G's IAW FRP-H.1 is AFW, _____ (1) _____.

- Subsequently, APP-006-E6, CST LO/LO-LO LEVEL alarms

The specific steps to reposition valves to align Service Water as a backup to the AFW system are located in _____ (2) _____.

OP-402, AUXILIARY FEEDWATER SYSTEM
FRP-H.1, ATTACHMENT 1, ALIGNING ALTERNATE WATER SUPPLY TO AFW PUMPS

- A. (1) Main Feedwater, Condensate
(2) OP-402
 - B. (1) Main Feedwater, Condensate
(2) ATTACHMENT 1 of FRP-H.1
 - C. (1) Condensate, Main Feedwater
(2) OP-402
 - D. (1) Condensate, Main Feedwater
(2) ATTACHMENT 1 of FRP-H.1
-

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ILC-17 RNP SRO NRC Examination

Question: 82
(1 point)

Given the following:

- The plant is stabilized at 3% power with GP-003, NORMAL PLANT STARTUP FROM HOT SHUTDOWN TO CRITICAL, in progress
- The BOP is performing OWP-011, NUCLEAR INSTRUMENTATION (NI), for Intermediate Range Channel N-35 to support I&C Troubleshooting activities
- When the LEVEL TRIP switch for N-35 is positioned to BYPASS, the BOP observes the indications of a blown fuse on the N-35 drawer
- The reactor remains at 3% power
- The CRS has entered Condition F of LCO 3.3.1, REACTOR PROTECTION SYSTEM (RPS) INSTRUMENTATION

Which ONE of the following correctly completes the statements below?

The ____ (1) ____ fuse is blown.

IAW Technical Specification, the CRS ____ (2) ____ allowed to enter MODE 1.

REFERENCE PROVIDED

- A. (1) Control Power
(2) is
 - B. (1) Control Power
(2) is NOT
 - C. (1) Instrument Power
(2) is NOT
 - D. (1) Instrument Power
(2) is
-

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ILC-17 RNP SRO NRC Examination

Question: 83
(1 point)

Given the following:

- "A" WGDT is IN SERVICE
- Chemistry informs the CRS that as a result of a fuel element failure, "A" WGDT activity is 2.6×10^4 Curies Noble Gas

Subsequently, vent header pressure is 2.3 PSIG

Which ONE of the following correctly completes the statements below?

The CRS will direct vent header pressure to be reduced _____ (1) _____ the "A" WGDT is removed from service.

ODCM 3.2.1 states that the dose rate due to radioactive materials in gaseous effluents released from the site boundary shall be limited to the following: For radionoble gases, _____ (2) _____ to total body.

- A. (1) AFTER
(2) ≤ 300 mrem/yr
 - B. (1) BEFORE
(2) ≤ 300 mrem/yr
 - C. (1) AFTER
(2) ≤ 500 mrem/yr
 - D. (1) BEFORE
(2) ≤ 500 mrem/yr
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 84
(1 point)

Given the following:

- Core Alterations are on-going in accordance with GP-010, REFUELING
- R-2, CV Low Range, alarms (HIGH LED is LIT)

Which ONE of the following correctly completes the statements below?

To determine the correct R-2 setpoint the operator is required to use _____ (1).

- R-2 was subsequently determined to be inoperable
- An HP Technician is providing continuous coverage with a Portable Area Radiation Monitor

Core Alterations _____ (2) continue.

- A. (1) the Setpoint Change and Log Record of OMM-014, RADIATION MONITOR SETPOINTS
(2) can
- B. (1) the Setpoint Change and Log Record of OMM-014, RADIATION MONITOR SETPOINTS
(2) can NOT
- C. (1) Section 3.1, Monitor Alarm Setpoint Determination, of the OFFSITE DOSE CALCULATION MANUAL (ODCM)
(2) can
- D. (1) Section 3.1, Monitor Alarm Setpoint Determination, of the OFFSITE DOSE CALCULATION MANUAL (ODCM)
(2) can NOT
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 85
(1 point)

Given the following:

- The crew is in EOP-E-1, LOSS OF REACTOR OR SECONDARY COOLANT
- Containment pressure is 5 psig and slowly rising
- RCS pressure is 1000 psig and stable
- RCS temperature is 508°F and slowly lowering
- Total AFW flow is 650 gpm
- PZR level is 32% and stable
- RWST Level is 60% and slowly lowering
- CCW Pump "B" is OOS
- CCW Pump "C" has tripped

Which ONE of the following correctly completes the statements below?

IAW EOP-E-1, the NEXT require procedure transition is to _____.

- A. EOP-ES-1.1, SI TERMINATION
 - B. EOP-ES-1.3, TRANSFER TO COLD LEG RECIRCULATION
 - C. EOP-ES-1.2, POST LOCA COOLDOWN AND DEPRESSURIZATION
 - D. EOP-ECA-1.1, LOSS OF EMERGENCY COOLANT RECIRCULATION
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 86
(1 point)

Given the following:

- The plant is operating at 100% RTP

| TME | EVENT |
|----------------|--|
| 0800 yesterday | The feeder breaker from Emergency Bus E-1 to MCC-5 tripped A loss of Instrument Bus 1 (IB-1) occurred AOP-024, Loss of Instrument Bus, entered |
| 0815 yesterday | WCC reports feeder breaker repair will take an extended amount of time |
| 0820 yesterday | Auxiliary Operator transferred IB-1 to MCC-8 |
| 0910 yesterday | Auxiliary Operator supplied power to MCC-5 from the DS bus |
| 0100 today | Auxiliary Operator transferred IB-1 from MCC-8 back to MCC-5 |
| 0805 today | Feeder breaker from Emergency Bus E-1 repaired. Power restored to MCC-5 from E-1 |

Which ONE of the following correctly completes the statements below?

The Auxiliary Operator transfers IB-1 power from MCC-8 back to MCC-5 in the
 (1)

ALL Technical Specifications REQUIRED ACTION COMPLETION TIME(s)
 (2) satisfied.

REFERENCE PROVIDED

- A. (1) Unit 2 Cable Spread Room
(2) were
- B. (1) Unit 2 Cable Spread Room
(2) were NOT
- C. (1) E1 / E2 Room
(2) were NOT
- D. (1) E1 / E2 Room
(2) were

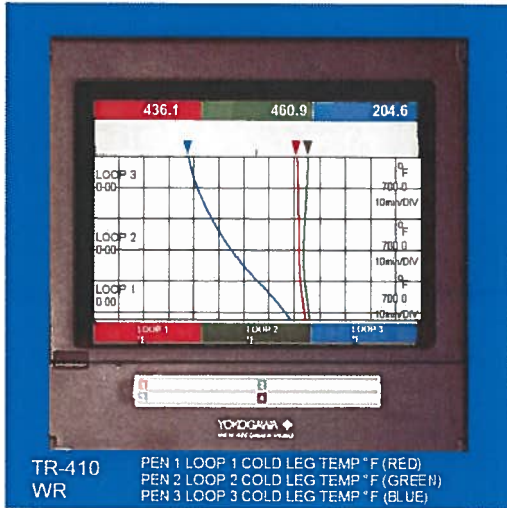
Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 87
(1 point)

Given the following:

- A reactor trip and safety injection were required due to a 'C' SGTR event
- Coincident with the plant trip, a loss of the Startup Transformer occurred
- The crew is in EOP-E-3, STEAM GENERATOR TUBE RUPTURE
- The crew has just stopped the initial RCS cooldown at the target temperature
- The OATC reports that CSFST-4, INTEGRITY is RED and observes the following:



Which ONE of the following identifies how the CRS is required to proceed, AND the reason for this action?

FRP-P.1, RESPONSE TO IMMINENT PRESSURIZED THERMAL SHOCK

- A. The CRS is required to immediately transition to FRP-P.1; because the RED path on CSF-4 is VALID and the mitigation strategy in FRP-P.1 is of higher priority than EOP-E-3
- B. The CRS is NOT required to immediately transition to FRP-P.1; because the RED path on CSF-4 is FALSE due to actions taken in EOP-E-3; later steps will check for a VALID CSF-4 RED path
- C. The CRS is required to immediately transition to FRP-P.1; because the RED path on CSF-4 is FALSE due to actions taken in EOP-E-3; based on the events in progress, early steps in FRP-P.1 will return the crew to procedure and step in effect
- D. The CRS is NOT required to immediately transition to FRP-P.1; because the RED path on CSF-4 is FALSE due to actions taken in EOP-E-3; it will ONLY be addressed when a transition out of EOP-E-3 to a subsequent Post-SGTR procedure occurs

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Question: 89
(1 point)

Given the following:

- A plane crashed into the Intake Structure
- The plant was manually tripped
- The SUT tripped
- The CRS has entered EPP-28, LOSS OF ULTIMATE HEAT SINK

ATTACHMENT 6, DEEPWELL COOLING

ATTACHMENT 7, ESTABLISHING CCW COOLING

Which ONE of the following correctly completes the statements below?

The **MAXIMUM** time the crew has to restore cooling to an EDG to preclude adverse effects is (1) minutes.

In order to provide cooling to at least one EDG, the CRS is required to direct ATTACHMENT (2) to be performed.

- A. (1) 60
(2) 6
 - B. (1) 60
(2) 7
 - C. (1) 40
(2) 6
 - D. (1) 40
(2) 7
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 90
(1 point)

Given the following:

- The plant is at 100% RTP
- Containment Purge is in service to reduce CV temperature for CV entry
- R-12 is in alarm and confirmed to be VALID
- R-12 is reading lower than the release permit allowed level

Which ONE of the following correctly completes the statements below?

An NRC report ____ (1) ____ required IAW AP-030, NRC REPORTING REQUIREMENTS.

The CV purge ____ (2) ____ be restarted under the CURRENT permit.

REFERENCE PROVIDED

- A. (1) is
(2) may
 - B. (1) is
(2) may NOT
 - C. (1) is NOT
(2) may
 - D. (1) is NOT
(2) may NOT
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 91
(1 point)

Given the following:

- GP-003, NORMAL PLANT STARTUP FROM HOT SHUTDOWN TO CRITICAL, is in progress
- The reactor is subcritical
- The OATC is in the process of bringing the reactor critical
- The licensed operator performing Attachment 10.3, CONTROL ROD WITHDRAWAL CHECKLIST, of GP-003 reports that Bank D Control Rod Group Step Counters indicate 1 Step when Bank C Group Step Counters indicate 128 steps

Which ONE of the following correctly completes the statements below?

Rod Control Bank Overlap ensures acceptable ____ (1) ____.

IAW Technical Specification LCO 3.0.4, the reactor startup ____ (2) ____ continue.

- A. (1) shutdown margin
(2) can NOT
 - B. (1) shutdown margin
(2) can
 - C. (1) power peaking during control bank rod motion
(2) can NOT
 - D. (1) power peaking during control bank rod motion
(2) can
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 92
(1 point)

Given the following:

- The plant is in Mode 6 with a core offload in progress
- The CV Manipulator is moving a fuel assembly from core location H-8 to the CV upender
- Before the fuel assembly is lowered into the CV upender, the CV manipulator operator accidentally places the gripper switch to the "DISENGAGE" position

Subsequently;

- The fuel assembly releases from the gripper while in the full up position and lands on the CV upender

Which ONE of the following correctly completes the statements below?

The fuel assembly dropped due to a failure of _____ (1) _____.

IAW OMM-001-18, OUTAGES, The refueling SRO _____ (2) _____ authorized to approve bypassing refueling interlocks.

- A. (1) ONLY the "Gripper Lock" Interlock
(2) is
 - B. (1) ONLY the "Gripper Lock" Interlock
(2) is NOT
 - C. (1) BOTH the "Gripper-Weight Indicator" AND the "Gripper Lock" Interlocks
(2) is
 - D. (1) BOTH the "Gripper-Weight Indicator" AND the "Gripper Lock" Interlocks
(2) is NOT
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 93
(1 point)

Given the following:

- The plant is at 400°F during a plant startup
- LT-474, Steam Generator 'A' Level Channel, has been removed from service for maintenance per plant procedure.

Which ONE of the following identifies ALL the Tech Spec Limiting Conditions for Operation (LCO(s)) that have required action(s) due to these conditions?

REFERENCE PROVIDED

- A. LCO 3.3.1, ONLY
 - B. LCO 3.3.8, ONLY
 - C. LCO 3.3.1 and LCO 3.3.3, ONLY
 - D. LCO 3.3.3 and LCO 3.3.8, ONLY
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 94
(1 point)

Given the following:

- At 09:00 an ALERT Emergency Classification was declared
- At 09:10 conditions degraded and a SITE AREA EMERGENCY was declared

The **LATEST** time that the **INITIAL** notification to the State and County Emergency Organizations is required is (1) .

- At 10:00 conditions degraded and a GENERAL EMERGENCY was declared

The **LATEST** time that the **NRC** is required to be notified of the upgrade to GENERAL EMERGENCY is (2) .

- A. (1) 0915
(2) 1015
 - B. (1) 0925
(2) 1015
 - C. (1) 0915
(2) 1100
 - D. (1) 0925
(2) 1100
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 95
(1 point)

Which ONE of the following correctly completes the statements below?

IAW FHP-003, FUEL ASSEMBLY MOVEMENT IN THE SPENT FUEL PIT, a Refueling SRO ____ (1) ____ required to be established to load new fuel into the Spent Fuel Pool.

IAW FHP-003, permission is required from ____ (2) ____ to turn on the lights within the Spent Fuel Pool in support of spent fuel movement within the Spent Fuel Pool.

- A. (1) is
(2) the WCC SRO
 - B. (1) is
(2) Reactor Engineering
 - C. (1) is NOT
(2) the WCC SRO
 - D. (1) is NOT
(2) Reactor Engineering
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 96
(1 point)

Given the following:

An outage is planned in which the plant is to be taken from MODE 1 to a complete core off-load into the Spent Fuel Pit.

Which ONE of the following correctly completes the statements below?

The EARLIEST that the requirements of OMP-003, SHUTDOWN SAFETY FUNCTION GUIDELINES, must be implemented is when ____ (1) ____ is entered.

The requirements of OMP-003 ____ (2) ____ when the core is completely off-loaded to the Spent Fuel Pit.

- A. (1) MODE 3
(2) do NOT apply
 - B. (1) MODE 3
(2) apply
 - C. (1) MODE 4
(2) apply
 - D. (1) MODE 4
(2) do NOT apply
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 97
(1 point)

Given the following:

- The plant is operating at 100% power
- The crew performed OST-151-1, SAFETY INJECTION SYSTEM COMPONENTS TEST – PUMP A with the following results:
 - On the initial stroke of SI-869, LOOPS “B” AND “C” HOT LEG INJECTION SHUTOFF, the stroke time exceeded the CODE CRITERIA, but was less than the LIMITING VALUE
 - During the run of the A SI Pump the operator recorded vibration level in the REQUIRED ACTION range

Subsequently, the following sequence of events occur:

- The A SI Pump is shutdown, placed under clearance, and repairs completed
- The A SI Pump is retested IAW OST-151-1 and vibrations are now in the ALERT range

IAW OMM-015, OPERATIONS SURVEILLANCE TESTING, which ONE of the following correctly completes the statements below?

For SI-869, OMM-015 (1) allow the operators to immediately retest and evaluate SI-869 using the process defined in OMM-015.

Based on the retest, the A SI Pump is (2) .

- A. (1) does
(2) OPERABLE
 - B. (1) does NOT
(2) OPERABLE
 - C. (1) does
(2) inoperable
 - D. (1) does NOT
(2) inoperable
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 98
(1 point)

Which ONE of the following correctly completes the statement below?

IAW LCO 3.3.3. Bases, radiation monitor(s) _____ is/are required to be OPERABLE to monitor for the potential of significant radiation releases, to provide release assessment for use by operators in determining the need to invoke site emergency plans, and to determine the type of high energy line break that has occurred inside Containment.

- A. R-2, Containment
 - B. R-11, Containment Particulate
 - C. R-12, Containment Gas
 - D. R-32A/B, Containment High Range
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 99
(1 point)

Given the following:

- A reactor trip and safety injection has occurred from 100% RTP

Which ONE of the following correctly completes the statements below?

IAW OMM-022, EMERGENCY OPERATING PROCEDURES USER'S GUIDE...

another licensed operator implements concurrent AOPs after EOP-E-0 immediate actions (1).

AOP-034 and AOP-041 (2) apply during the performance of RED or ORANGE FRPs.

AOP-034, SECURITY EVENTS
AOP-041, RESPONSE TO FIRE EVENT

- A. (1) have been completed and verified, ONLY
(2) do
 - B. (1) have been completed and verified, ONLY
(2) do NOT
 - C. (1) AND Attachment 1 of EOP-E-0 have been completed
(2) do
 - D. (1) AND Attachment 1 of EOP-E-0 have been completed
(2) do NOT
-

Robinson Nuclear Plant

ILC-17 RNP SRO NRC Examination

Question: 100

(1 point)

Which ONE of the following procedures does NOT contain a transition to SACRM-1, SEVERE ACCIDENT CONTROL ROOM MANAGEMENT – INITIAL RESPONSE?

- A. EOP-ECA-0.0, LOSS OF ALL AC POWER
 - B. FRP-C.1, RESPONSE TO INADEQUATE CORE COOLING
 - C. FRP-H.1, RESPONSE TO LOSS OF SECONDARY HEAT SINK
 - D. FRP-S.1, RESPONSE TO NUCLEAR POWER GENERATION - ATWS
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Examination KEY for: ILC-17 RNP SRO NRC Exami

| <i>Question Number</i> | <i>Answer</i> |
|----------------------------|---------------|
| 76 | A |
| 77 | A |
| 78 | B |
| 79 | C |
| 80 | B |
| 81 | A |
| 82 | D |
| 83 | C |
| 84 | A |
| 85 | C |
| 86 | C |
| 87 | B |
| 88 | B |
| 89 | C |
| 90 | A |
| 91 | C |
| 92 | D |
| 93 | B |
| 94 | C |
| 95 | C |
| 96 | C |
| 97 | A |
| 98 | D |
| 99 | C |
| 100 | C |

2017 H. B. ROBINSON NRC SRO Written Exam

Additional References Provided

In addition to the references provided for the RO Written Exam, SRO applicants were also provided the following material on the day of examination:

1. Redacted copy of the “hot” EAL chart.
2. Redacted page 3.3-3 of Technical Specifications.
3. Non-redacted pages 3.8-19 through 3.8-34 of Technical Specifications.
4. Attachment 1, “Immediate (One-Hour) Notifications to the NRC,” of AP-030, “NRC REPORTING REQUIREMENTS,” with page 1 of 11 intentionally deleted.
5. Attachment 2, “Four Hour Notification to the NRC,” of AP-030.
6. Attachment 3, “Eight Hour Notifications to the NRC,” of AP-030.
7. Redacted page 3.3-15 of Technical Specifications.
8. Redacted page 3.3-32 of Technical Specifications.
9. Redacted page 3.3-47 of Technical Specifications.