



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 21, 1997

Mr. Thomas F. Plunkett
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

50-250

SUBJECT: COMPLETION OF LICENSING ACTION FOR GENERIC LETTER 95-03,
"CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES," FOR
TURKEY POINT UNITS 3 AND 4 AND ST. LUCIE UNITS 1 AND 2
(TAC NOS. M92284, M92285, M92278, AND M92279)

Dear Mr. Plunkett:

On April 28, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 95-03, "Circumferential Cracking of Steam Generator Tubes," to all holders of operating licenses or construction permits for pressurized-water reactors. The NRC issued GL 95-03 for three principal reasons:

- (1) Notify addressees about the safety significance of the recent steam generator tube inspection findings at Maine Yankee Atomic Power Station.
- (2) Request that all addressees implement the actions described within the generic letter.
- (3) Require that all addressees submit to the NRC a written response regarding implementation of the requested actions.

In addition, GL 95-03 alerted addressees to the importance of performing comprehensive examinations of steam generator tubes using techniques and equipment capable of reliably detecting the types of degradation to which the steam generator tubes may be susceptible. The staff also noted that the performance of steam generator tube examinations is controlled, in part, by Appendix B to Title 10, Part 50, of the *Code of Federal Regulations* (10 CFR Part 50).

In GL 95-03, the NRC staff also requested that licensees take the following actions:

- (1) Evaluate recent operating experience with respect to the detection and sizing of circumferential indications to determine the applicability to their plants.
- (2) On the basis of the evaluation in Item (1) above, as well as past inspection scope and results, susceptibility to circumferential cracking, threshold of detection, expected or inferred crack growth rates, and other relevant factors, develop a safety assessment justifying continued operation until the next scheduled steam generator tube inspections.

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Mr. T. Plunkett

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- (3) Develop plans for the next steam generator tube inspections as they pertain to the detection of circumferential cracking. The inspection plans should address, but not be limited to, scope (including sample expansion criteria, if applicable), methods, equipment, and criteria (including personnel training and qualification).

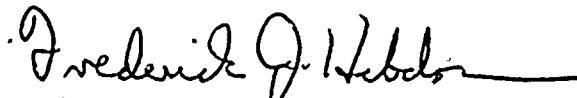
To document the outcome of these actions, the NRC staff requested that addressees prepare and submit the following:

- (1) A safety assessment justifying continued operation, predicated on the evaluation performed in accordance with requested actions 1 and 2 (above).
- (2) A summary of the inspection plans developed in accordance with requested action 3 (above) and a schedule for the next planned inspection.

In response to GL 95-03, you provided letters dated June 22, 1995, October 11, 1995, January 16, 1996, and February 27, 1996 for Turkey Point Units 3 and 4 and June 23, 1995 and December 11, 1995 for St. Lucie Units 1 and 2. These submittals provided the information requested by GL 95-03; therefore TAC Nos. M92284, M92285, M92278, AND M92279 are closed. For your information, the staff's findings regarding this issue are contained in NUREG-1604, "Circumferential Cracking of Steam Generator Tubes."

If you have any questions regarding this matter, please contact Richard Croteau at (301) 415-1475.

Sincerely,



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-335, 50-389, 50-250 and 50-251

cc: See next page



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(3) Develop plans for the next steam generator tube inspections as they pertain to the detection of circumferential cracking. The inspection plans should address, but not be limited to, scope (including sample expansion criteria, if applicable), methods, equipment, and criteria (including personnel training and qualification).

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Original signed by
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Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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NAME	RCroteau <i>RC</i>	BClayton <i>BC</i>	LWiens <i>LW</i>	FHebdon <i>FH</i>
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Mr. T. F. Plunkett
Florida Power and Light Company

cc:

M. S. Ross, Attorney
Florida Power & Light Company
P.O. Box 14000
Juno Beach, FL 33408-0420

John T. Butler, Esquire
Steel, Hector and Davis
4000 Southeast Financial Center
Miami, Florida 33131-2398

Mr. Robert J. Hovey, Site
Vice President
Turkey Point Nuclear Plant
Florida Power and Light Company
9760 SW. 344th Street
Florida City, FL 33035

Armando Vidal
County Manager
Metropolitan Dade County
111 NW 1 Street, 29th Floor
Miami, Florida 33128

Senior Resident Inspector
Turkey Point Nuclear Generating
Station
U.S. Nuclear Regulatory Commission
P.O. Box 1448
Homestead, Florida 33090

Mr. Bill Passetti
Office of Radiation Control
Department of Health and
Rehabilitative Services
1317 Winewood Blvd.
Tallahassee, Florida 32399-0700

TURKEY POINT PLANT

Mr. Joe Myers, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, Florida 32399-2100

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
61 Forsyth Street, SW., Suite 23T85
Atlanta, GA 30303-3415

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Plant Manager
Turkey Point Nuclear Plant
Florida Power and Light Company
9760 SW. 344th Street
Florida City, FL 33035

Mr. H.N. Paduano, Manager
Licensing & Special Programs
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Mr. Gary E. Hollinger
Licensing Manager
Turkey Point Nuclear Plant
9760 SW. 344th Street
Florida City, FL 33035

Mr. Kerry Landis
U.S. Nuclear Regulatory Commission
61 Forsyth Street, SW., Suite 23T85
Atlanta, GA 30303-3415

