



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO THE THIRD TEN-YEAR INTERVAL INSERVICE INSPECTION PROGRAM PLAN
AND ASSOCIATED REQUESTS FOR RELIEF
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

The Technical Specifications for Turkey Point Nuclear Power Plant, Units 3 and 4, state that the inservice inspection of the American Society of Mechanical Engineers (ASME) Code Class 1, 2, and 3 components shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i). 10 CFR 50.55a(a)(3) states that alternatives to the requirements of paragraph (g) may be used, when authorized by the NRC, if (i) the proposed alternatives would provide an acceptable level of quality and safety or (ii) compliance with the specified requirements would result in hardship or unusual difficulties without a compensating increase in the level of quality and safety.

Pursuant to 10 CFR 50.55a(g)(4), ASME Code Class 1, 2, and 3 components (including supports) shall meet the requirements, except the design and access provisions and the preservice examination requirements, set forth in the ASME Code, Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components," to the extent practical within the limitations of design, geometry, and materials of construction of the components. The regulations require that inservice examination of components and system pressure tests conducted during the first 10-year interval and subsequent intervals comply with the requirements in the latest edition and addenda of Section XI of the ASME Code incorporated by reference in 10 CFR 50.55a(b) on the date 12 months prior to the start of the 120-month interval, subject to the limitations and modifications listed therein. The applicable edition of Section XI of the ASME Code for the Turkey Point Nuclear Power Plant, Units 3 and 4, Third 10-year inservice inspection (ISI) interval is the 1989 Edition. The components (including supports) may meet the requirements set forth in subsequent editions and addenda of the ASME Code incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein and subject to Commission approval.

9504040024 950331
PDR ADOCK 05000250
G PDR



11

11

11

11

11

11

11

Pursuant to 10 CFR 50.55a(g)(5), if the licensee determines that conformance with an examination requirement of Section XI of the ASME Code is not practical for its facility, information shall be submitted to the Commission in support of that determination and a request made for relief from the ASME Code requirement. After evaluation of the determination, pursuant to 10 CFR 50.55a(g)(6)(i), the Commission may grant relief and may impose alternative requirements that are determined to be authorized by law, will not endanger life, property, or the common defense and security, and are otherwise in the public interest, giving due consideration to the burden upon the licensee that could result if the requirements were imposed. In a letter dated September 9, 1993, Florida Power and Light Company submitted to the NRC its Third Ten-Year ISI Program Plan and associated requests for relief for the Turkey Point Nuclear Power Plant, Units 3 and 4. Additional information was provided by the licensee in its letter dated May 31, 1994.

2.0 EVALUATION AND CONCLUSIONS

The staff, with technical assistance from its contractor, the Idaho National Engineering Laboratory (INEL), has evaluated the information provided by the licensee in support of its Third Ten-Year Interval Inservice Inspection (ISI) Program Plan and associated Requests for Relief for Turkey Point Nuclear Power Plant, Units 3 and 4. Based on the information submitted, the staff adopts the contractor's conclusions and recommendations presented in the Technical Evaluation Report (TER) with the exception of Requests for Relief Nos. RR-02, RR-09, and RR-10. With respect to RR-02, the contractor's TER recommends authorization of the licensee's proposed alternative provided the licensee demonstrate that the ultrasonic technique is capable of detecting circumferentially oriented outside surface flaws. As an additional condition for the staff to authorize the licensee's alternative of performing an inside surface ultrasonic examination in lieu of an outside surface examination, the ultrasonic technique must be capable of detecting and sizing circumferentially and axially oriented surface connected flaws such that no flaw accepted for continued service could exceed the Code allowable crack dimension before the next scheduled examination. NRC approval of this demonstration is not required, but the demonstration may be subject to inspection. The licensee has committed to provide additional information regarding Requests for Relief Nos. 09 and 10. The two requests will be evaluated in a separate report. Based on the contractor's review of the subject ISI program, no deviations from regulatory requirements or commitments were identified in the *Turkey Point Nuclear Power Plant, Units 3 and 4, Third Ten-Year Interval ISI Program Plan*.

Requests for Relief Nos. RR-01, RR-03 (with the exception of the of the terminal ends of piping at the inlets and outlets of RHX assembly), and RR-13 are granted pursuant to 10 CFR 50.55a(g)(6)(i). The alternative contained in Request for Relief No. RR-07 (Parts 1 and 2), is authorized pursuant to 10 CFR 50.55a(a)(3)(i). Request for Relief No. RR-6 is denied and RR-04 will be evaluated by others in a separate report.

The alternatives contained in Requests for Relief Nos. RR-02, RR-05, RR-08, RR-11, and RR-12 are authorized pursuant to 10 CFR 50.55a(a)(3)(i) with conditions. The proposed alternative contained in RR-02 is authorized with the additional condition noted above. The proposed alternative contained in RR-05 is authorized provided that the Category B-D reactor pressure vessel nozzles examinations are no more than 10 years (as defined in paragraph IWA-2430) between examinations. The proposed alternative in RR-08 is authorized provided that a 10% sample of integral attachments in each Code Class is scheduled for examination. The proposed alternative contained in RR-11 is authorized provided the licensee remove at least one bolt, closest to the source of leakage, and perform a VT-1 visual examination as part of the evaluation. The proposed alternative contained in RR-12 is authorized provided that the licensee meets the requirements of IWA-5260 for any pressure tests performed at pressures or temperatures exceeding nominal operating pressure.

Principal Contributor: T. McLellan

Date: March 31, 1995



Handwritten marks and symbols in the top right corner, including a large 'A' and several smaller characters.

Vertical text on the left side of the page, possibly a page number or reference code.

Small text or markings near the top center of the page.

Vertical text on the left side of the page, possibly a page number or reference code.

Small text or markings near the bottom center of the page.