



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 REGION II
 101 MARIETTA STREET, N.W., SUITE 2900
 ATLANTA, GEORGIA 30323

Report Nos.: 50-250/92-18 and 50-251/92-18

Licensee: Florida Power and Light Company
 9250 West Flagler Street
 Miami, FL 33102

Docket Nos.: 50-250 and 50-251 License Nos.: DPR-31 and DPR-41

Facility Name: Turkey Point Units 3 and 4

Inspection Conducted: July 25 through August 22, 1992

Inspectors:

<u><i>R. C. Butcher</i></u>	<u>9/14/92</u>
R. C. Butcher, Senior Resident Inspector	Date Signed
<u><i>G. A. Schnebli</i></u>	<u>9/14/92</u>
G. A. Schnebli, Resident Inspector	Date Signed
<u><i>L. Trocine</i></u>	<u>9/14/92</u>
L. Trocine, Resident Inspector	Date Signed

Accompanying Personnel: M. T. Janus

Approved by:

<u><i>K. D. Landis</i></u>	<u>9/14/92</u>
K. D. Landis, Chief Reactor Projects Section 2B Division of Reactor Projects	Date Signed

SUMMARY

Scope:

This routine resident inspector inspection involved direct inspection at the site in the areas of followup on items of noncompliance, monthly surveillance observations, monthly maintenance observations, engineered safety features walkdowns, operational safety verification, and preparation for refueling. Backshift inspections were performed on July 30, 1992.

Results:

Within the scope of this inspection, the inspectors determined that the licensee continued to demonstrate satisfactory performance to ensure safe plant operations.

REPORT DETAILS

1. Persons Contacted

Licensee Employees

T. V. Abbatiello, Site Quality Manager
R. J. Earl, Quality Assurance Supervisor
R. J. Gianfrennesco, Support Services Supervisor
E. F. Hayes, Instrumentation and Controls Maintenance Supervisor
R. G. Heisterman, Mechanical Maintenance Supervisor
D. E. Jernigan, Technical Manager
H. H. Johnson, Operations Supervisor
V. A. Kaminskis, Operations Manager
*J. E. Knorr, Regulatory Compliance Analyst
*R. S. Kundalkar, Engineering Manager
J. D. Lindsay, Health Physics Supervisor
G. L. Marsh, Reactor Engineering Supervisor
*L. W. Pearce, Plant General Manager
M. O. Pearce, Electrical Maintenance Supervisor
*T. F. Plunkett, Site Vice President
D. R. Powell, Services Manager
R. N. Steinke, Chemistry Supervisor
F. R. Timmons, Security Supervisor
M. B. Wayland, Maintenance Manager
J. D. Webb, Outage Manager (Acting)
*E. J. Weinkam, Licensing Manager

Other licensee employees contacted included construction craftsman, engineers, technicians, operators, mechanics, and electricians.

NRC Resident Inspectors

*R. C. Butcher, Senior Resident Inspector
G. A. Schnebli, Resident Inspector
L. Trocine, Resident Inspector

Accompanying NRC Inspector

M. T. Janus

Other NRC Personnel on Site

H. N. Berkow, NRR Project Directorate II-2-Director
L. Ragahavan, NRR Licensing Project Manager (Acting)

*Attended exit interview on August 28, 1992

Note: An alphabetical tabulation of acronyms used in this report is listed in the last paragraph in this report.



2. Plant Status

Unit 3

At the beginning of this reporting period, Unit 3 was operating at 87% power in order to extend the unit run time until the beginning of the August 24, 1992, refueling outage. Unit 3 had been on line since May 12, 1992. The following evolutions occurred on this unit during this assessment period:

- On July 29, 1992, at 9:30 p.m., a load reduction to approximately 70% power was commenced in order to extend the unit run time until the beginning of the August 24, 1992, refueling outage. Unit 3 reached 70% power at 12:30 p.m. on the following day, and reduced power operation was continued.
- On August 13, 1992, at 8:00 p.m. a load reduction to 60% power was commenced in order to facilitate the cleaning of the Unit 3 waterboxes. Reactor power was stabilized at 59% at 10:00 p.m. on the same day. The unit was returned to 70% power at 2:00 a.m. on August 15, 1992.

Unit 4

At the beginning of this reporting period, Unit 4 was operating at 100% power and had been on line since May 7, 1992. The following evolutions occurred on this unit during this assessment period:

- On August 6, 1992, at 6:50 p.m., a load reduction to approximately 40% power was commenced in order to facilitate turbine valve testing and TPCW heat exchanger cleaning. Reactor power was stabilized at about 40% by approximately 9:10 p.m. on the same day.
- On August 6, 1992, at 11:15 p.m., power ascension from approximately 42% power to 60% power was commenced, and the unit was stabilized at 66% power at 2:30 a.m. on August 7, 1992.
- On August 7, 1992, at 6:55 p.m., power ascension to 100% was commenced, and 100% power was achieved at 10:50 p.m. on the same day.
- On August 8, 1992, at 8:35 a.m., a load reduction to 75% power was commenced following the identification of tube leaks in the 3B feedwater heater. Unit 4 was stabilized at approximately 75% power at 9:05 a.m. on the same day.
- On August 8, 1992, at 5:05 p.m., a load reduction to approximately 60% power was commenced in order to facilitate the removal of the 5B feedwater heater from service, and the unit was stabilized at approximately 60% power at 5:45 p.m. on the same day.

- On August 9, 1992, at 6:00 a.m., a power ascension to approximately 70% was commenced. Reactor power was stabilized at about 67% at approximately 11:40 a.m. on the same day. The unit was returned to 100% power at 4:05 p.m. on August 15, 1992.

3. Followup on Items of Noncompliance (92702)

A review was conducted of the following noncompliances to assure that corrective actions were adequately implemented and resulted in conformance with regulatory requirements. Verification of corrective action was achieved through record reviews, observation, and discussions with licensee personnel. Licensee correspondence was evaluated to ensure the responses were timely and corrective actions were implemented within the time periods specified in the reply.

(Closed) VIO 50-250,251/92-10-01, Failure to Follow a Procedure Resulting in the Removal of a Unit 4, Channel A, RWST Level Transmitter From Service During the Calibration of a Unit 3, Channel B, RWST Level Transmitter.

This event occurred on April 2, 1992; a Notice of Violation was issued on May 29, 1992; and the licensee responded to the violation by letter dated June 26, 1992. The licensee attributed this event to a lack of attention to detail and to the failure of an I&C technician to follow an approved procedure. Upon notification by the control room RCO of isolation of the wrong transmitter, the technician returned the transmitter to service under the direction of the Field supervisor. The Unit 4, Channel A, RWST level transmitter was returned to service within 10 minutes. The licensee counselled the technician on the need to use self checking techniques even when using procedures, and the technician was disciplined in accordance with FPL Nuclear Division policy. In addition, the licensee revised the Instrument and Control Maintenance Initial and Continuing Training Programs to incorporate the use of self checking (stop, locate, touch, verify, anticipate, manipulate, and observe). The inspectors have reviewed the licensee's corrective actions with regard to this event and have found them to be adequate. This issue is closed.

4. Monthly Surveillance Observations (61726)

The inspectors observed TS required surveillance testing and verified that the test procedures conformed to the requirements of the TSs; testing was performed in accordance with adequate procedures; test instrumentation was calibrated; limiting conditions for operation were met; test results met acceptance criteria requirements and were reviewed by personnel other than the individual directing the test; deficiencies were identified, as appropriate, and were properly reviewed and resolved by management personnel; and system restoration was adequate. For completed tests, the inspectors verified testing frequencies were met and tests were performed by qualified individuals.



The inspectors witnessed/reviewed portions of the following test activities:

- 0-PME-102.14, Thermography Inspection of 480V and 4KV Electrical Components, for Unit 3;
- 3-OSP-033.1, Spent Fuel Pit Cooling Pump Inservice Test;
- 3/4-OSP-075.9, AFW Overspeed Test, for the C AFW pump;
- 3-OSP-059.1, Source Range Nuclear Instrumentation Analog Channel Operational Test;
- 4-OSP-090.1, Main Generator Fuse Inspection; and
- 0-OSP-075.11, Auxiliary Feedwater Inservice Test, for the C AFW pump.

The inspectors determined that the above testing activities were performed in a satisfactory manner and met the requirements of the TSs. Violations or deviations were not identified.

5. Monthly Maintenance Observations (62703)

Station maintenance activities of safety-related systems and components were observed and reviewed to ascertain they were conducted in accordance with approved procedures, regulatory guides, industry codes and standards, and in conformance with the TSs.

The following items were considered during this review, as appropriate: LCOs were met while components or systems were removed from service; approvals were obtained prior to initiating work; activities were accomplished using approved procedures and were inspected as applicable; procedures used were adequate to control the activity; troubleshooting activities were controlled and repair records accurately reflected the maintenance performed; functional testing and/or calibrations were performed prior to returning components or systems to service; QC records were maintained; activities were accomplished by qualified personnel; parts and materials used were properly certified; radiological controls were properly implemented; QC hold points were established and observed where required; fire prevention controls were implemented; outside contractor force activities were controlled in accordance with the approved QA program; and housekeeping was actively pursued.

The inspectors witnessed/reviewed portions of the following maintenance activities in progress:

- Troubleshooting and repair of Unit 4 3B feedwater heater tube leaks,
- overhaul of the C AFW pump turbine, and

- troubleshooting and repair of the B AFW pump failed thrust bearing.

For those maintenance activities observed, the inspectors determined that the activities were conducted in a satisfactory manner and that the work was properly performed in accordance with approved maintenance work orders. Violations or deviations were not identified.

6. Engineered Safety Features Walkdown (71710)

The inspectors performed an inspection designed to verify the status of the Unit 3 and Unit 4 component cooling water systems located outside of containment. This was accomplished by performing a complete walkdown of all accessible equipment, utilizing plant procedures 3-OP-030 and 4-OP-030, Component Cooling Water System; 3-OSP-030.3 and 4-OSP-030.3, Component Cooling Water System Flowpath Verification; and plant drawing 5610-T-E-4512, Rev. 96, Operating Diagram Component Cooling System Outside Containment. The following criteria were used, as appropriate, during this inspection:

- systems lineup procedures matched plant drawings and as-built configuration;
- housekeeping was adequate, and appropriate levels of cleanliness were being maintained;
- valves in the system were correctly installed and did not exhibit signs of gross packing leakage, bent stems, missing handwheels, or improper labeling;
- hangers and supports were made up properly and aligned correctly;
- valves in the flow paths were in correct position as required by the applicable procedures with power available, and valves were locked/lock wired as required;
- local and remote position indication was compared, and remote instrumentation was functional; and
- major system components were properly labeled.

Some minor drawing and labeling discrepancies were identified and brought to the attention of the system engineer for correction.

Violations or deviations were not identified.

7. Operational Safety Verification (71707)

The inspectors observed control room operations, reviewed applicable logs, conducted discussions with control room operators, observed shift turnovers, and monitored instrumentation. The inspectors verified proper valve/switch alignment of selected emergency systems, verified



maintenance work orders had been submitted as required, and verified followup and prioritization of work was accomplished. The inspectors reviewed tagout records, verified compliance with TS LCOs, and verified the return to service of affected components.

By observation and direct interviews, verification was made that the physical security plan was being implemented. The implementation of radiological controls and plant housekeeping/cleanliness conditions were also observed.

Tours of the intake structure and diesel, auxiliary, control, and turbine buildings were conducted to observe plant equipment conditions including potential fire hazards, fluid leaks, and excessive vibrations.

The inspectors walked down accessible portions of the following safety-related systems/structures to verify proper valve/switch alignment:

- A and B emergency diesel generators,
- control room vertical panels and safeguards racks,
- intake cooling water structure,
- 4160-volt buses and 480-volt load and motor control centers,
- Unit 3 and 4 feedwater platforms,
- Unit 3 and 4 condensate storage tank area,
- auxiliary feedwater area,
- Unit 3 and 4 main steam platforms, and
- auxiliary building.

The licensee routinely performs QA/QC audits/surveillances of activities required under its QA program and as requested by management. To assess the effectiveness of these licensee audits, the inspectors examined the status, scope, and findings of the following audit reports:

<u>Audit Number</u>	<u>Number of Findings</u>	<u>Type of Audit</u>
QA0-PTN-92-021	1	Housekeeping/Special Processes, QP 2.8, 2.10, 9.1, 9.4, and 10.3
QA0-PTN-92-026	1	June Performance Monitoring Audit
QA0-PTN-92-027	-	Emergency Core Cooling System
QA0-PTN-92-028	-	Radwaste Handling and Shipping

No additional NRC followup actions will be taken on the findings referenced above because they were identified by the licensee's QA program audits and corrective actions have either been completed or are currently underway. Plant management has also been made aware of these issues.

As a result of routine plant tours and various operational observations, the inspectors determined that the general plant and system material conditions were satisfactorily maintained, the plant security program was effective, and the overall performance of plant operations was good.

Violations or deviations were not identified.

8. Preparation For Refueling (60705)

On July 24, 27, 31 and August 7 and 10, 1992, the inspectors witnessed portions of the off loading process for the third, fourth, and fifth shipments of new fuel received on site. This process included the transfer of the new fuel elements from the shipping containers to the new fuel room storage racks and the subsequent transfer from the new fuel storage racks to interim storage in the spent fuel pool for the August RFO. This final transfer to the SFP was done in accordance with the same procedures used for the receipt and transfer of the new fuel, O-OP-040.1, Handling New Fuel Shipping Containers and New Fuel Assemblies and O-OSP-040.11, Receipt of New Fuel. The process was supported by operations, quality assurance, health physics, security, and reactor engineering personnel, who handled and coordinated these receipt/transfer evolutions in a proficient manner.

Violations or deviations were not identified.

9. Exit Interview (30703)

The inspection scope and findings were summarized during management interviews held throughout the reporting period with the Plant General Manager and selected members of his staff. An exit meeting was conducted on September 1, 1992. The areas requiring management attention were reviewed. The licensee did not identify as proprietary any of the materials provided to or reviewed by the inspectors during this inspection. Dissenting comments were not received from the licensee. Violations or deviations were not identified.

11. Acronyms and Abbreviations

AFW	Auxiliary Feedwater
FPL	Florida Power & Light
I&C	Instrument and Controls
KV	Kilovolt
LCO	Limiting Condition for Operation
LER	Licensee Event Report
LOCA	Loss-of-Coolant Accident
NRC	Nuclear Regulatory Commission



OP	Operating Procedure
OSP	Operations Surveillance Procedure
PME	Preventive Maintenance-Electrical
PTN	Plant Turkey Nuclear
QA	Quality Assurance
QAO	Quality Assurance Organization
QC	Quality Control
QP	Quality Procedure
RCO	Reactor Control Operator
RFO	Refueling Outage
RWST	Refueling Water Storage Tank
SFP	Spent Fuel Pit
TPCW	Turbine Plant Cooling Water
TS	Technical Specification
V	Volt
VIO	Violation

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