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BUNE 5 1989

L-89-196 10 CFR 50.73

TP

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D. C. 20555

Gentlemen:

Turkey Point Unit 4 Re: Docket Nos. 50-251 Reportable Event: 89-03 Date of Event: May 5, 1989 Reactor Trip During Performance of Steam Generator Protection Set III Channel Test

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR 50.73 to provide notification of the subject event.

Very truly yours,

0. Woody

Acting Senior Vice President - Nuclear

COW/JRH/cm

Attachment

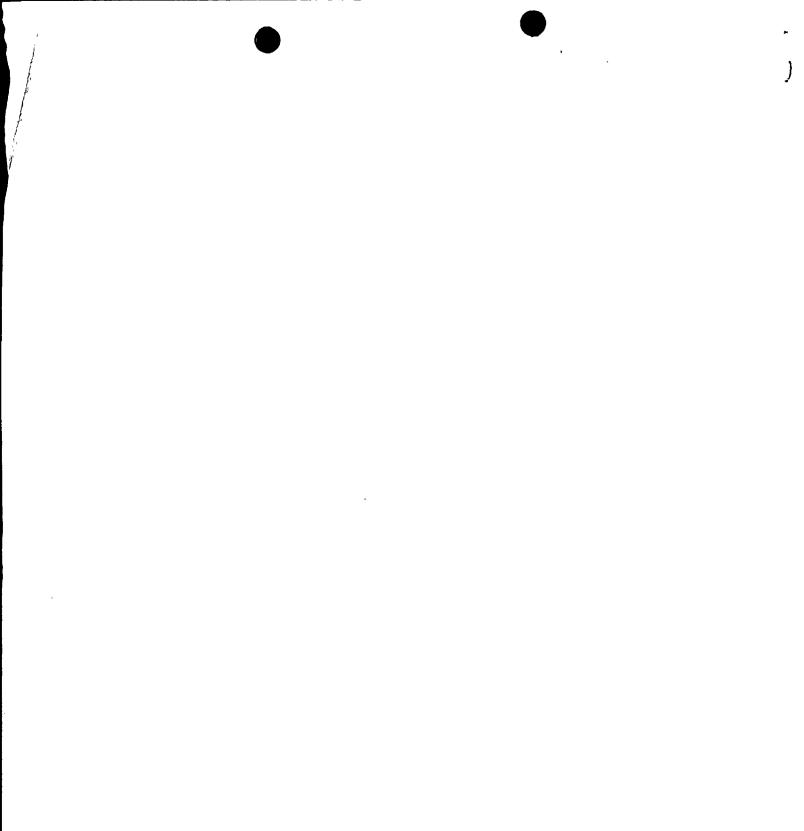
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Stewart D. Ebneter, Regional Administrator, Region II, USNRC cc: Senior Resident Inspector, USNRC, Turkey Point Plant

an FPL Group company

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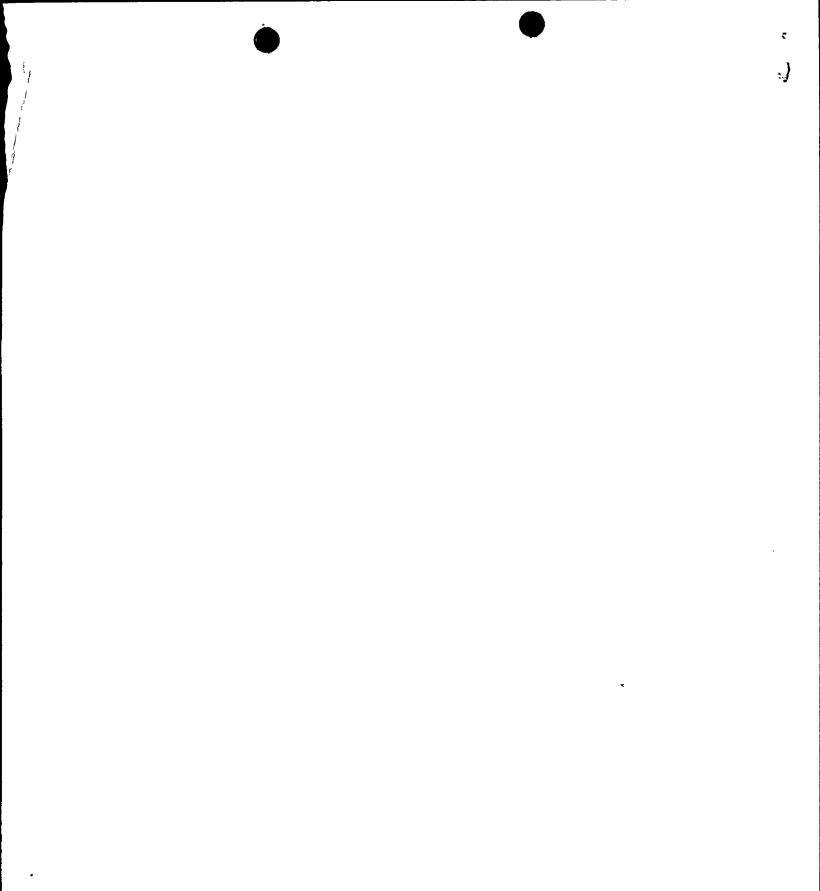


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### EVENT

On May 5, 1989, at 0152, with Unit 4 in Mode 3 (Hot Standby), a Reactor Protection System (RPS) (EIIS: JC) actuation occurred while performing procedure 4-SMI-071.4, "Steam Generator Protection Set III Analog Channel Test." At the time of occurrence, control banks C and D were withdrawn for rod drop testing in accordance with procedure 4-PMI-028.3, "RPI Hot Calibration, CRDM Stepping Test, and Rod Drop Test." The reactor tripped when Instrumentation and Control personnel (non-licensed utility personnel) placed bistable BS-4-446-1 in the Test position in accordance with step 6.2.3.1 of procedure 4-SMI-071.4: All systems functioned as designed, and Unit 4 remained stable in Mode 3 throughout the event. An investigation is being conducted to determine the root cause(s) of the ' incident and establish the corrective actions to prevent recurrence.

Placing BS-4-446-1 in the Test position simulated a reactor power greater than 10%, enabling the RPS low power permissive's (P-7) reactor trips; thus, one-half of the reactor trip logic was provided. A subsequent investigation determined that the coincident half of the reactor trip logic was provided by a turbine trip signal due to the presence of lifted leads in the turbine stop valve position sensing circuitry. The lifted leads prevented the RPS from receiving a signal indicating the turbine stop valves (EIIS:TA) were open. The turbine stop valves were verified to be in an open position.

### CAUSE OF EVENT

Our investigation into the event has not established the root cause of the event as of issuance of this report. A Supplemental Licensee Event Report will be issued to document the root cause(s) and corrective actions to prevent recurrence.

#### ANALYSIS OF EVENT

A post-trip review was conducted to assess the proper operation of safety-related equipment. The review established that no thermodynamic response to the trip occurred and that plant parameters responded as expected. Other than the automatic initiation of the reactor trip, there were no manual or automatic reactor protection system or engineered safety features actuations. Although the lifted leads prevented the RPS from receiving a true indication of the turbine stop valves position, the RPS was in the fail safe position indicating the turbine stop valves were closed. Based on the above, the health and safety of the public were not affected.

# CORRECTIVE ACTIONS

1) The lifted leads for the turbine stop valves' position sensing circuitry

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were landed on May 5, 1989.

2) An investigation is being conducted to establish the root cause(s) of the event and the corrective actions to prevent reoccurence. A Supplemental Licensee Event Report will be issued to document the root cause(s) of the event and the corrective actions to prevent reoccurrence.

## ADDITIONAL INFORMATION

Similar occurrences: LER 250-89-004 describes a reactor trip due to a defective procedure during performance of OP 14004.1 (the old procedure including the Steam Generator Protection Set III Analog Channel Test), LER 251-88-010 delineates a reactor trip due to personnel error during the performance of OP 14004.1, and LER 250-86-030 describes a reactor trip and safety injection due to inadequate planning of post-maintenance testing associated with OP 14004.1.

