

ENCLOSURE 1

NOTICE OF VIOLATION

Florida Power and Light Company
Turkey Point

Docket Nos. 50-250 and 50-251
License Nos. DPR-31 and DPR-41

During the Nuclear Regulatory Commission (NRC) inspection conducted on February 13-17, 1989, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," 10 CFR Part 2, Appendix C (1989), the violations are listed below:

- A. 10 CFR 50, Appendix B, Criteria V and the licensee's accepted Quality Assurance (QA) Program collectively require that activities affecting quality be prescribed by procedures and accomplished in accordance with those procedures. Procedures 3CMM-041.1, Pressurizer Safety Valve Repair and Setting, and 3SMM-041.1, Pressurizer Safety Valve Setpoint Testing, contain requirements for setting the ring settings for pressurizer safety valves.

Contrary to the above, activities affecting quality were not accomplished in accordance with procedures, in that, the pressurizer safety valve's as left ring settings were not adjusted to the as found ring settings following setpoint testing and maintenance during the Unit 3 1987 refueling outage.

This is a Severity Level IV violation (Supplement I).

- B. 10 CFR 50, Appendix B, Criteria VI and the licensee's accepted QA Program collectively require that measures be established to control the issuance of procedures, including changes which prescribe all activities affecting quality.

Contrary to the above, measures were not established to control changes to procedures, in that, pressurizer safety valve 551B was replaced during the Unit 4 1988 outage and governing procedures were not updated to reflect the new ring settings.

Contrary to the above, Florida Power and Light Inter-Office memorandum dated August 12, 1986, recommended changing similar procedures to reflect accurate pressurizer safety valve ring settings. This action was not accomplished prior to the Unit 3 refueling outage in 1987; consequently, the pressurizer safety valve ring settings were not set in accordance with the vendor as-shipped data.

This is a Severity Level IV violation (Supplement I).



- C. 10 CFR 50.55a.(g).(4).(ii) states that inservice examinations of components and inservice tests to verify operational readiness of pumps and valves whose function is required for safety shall comply with the requirements of the Code (ASME, Section XI).

ASME Boiler and Pressure Vessel Code - 1980, Section XI, Paragraph IWV-3417(a), requires that if a stroke time increase of 50 percent or more for valves with full-stroke times less than or equal to 10 seconds is observed, test frequency shall be increased to once each month until corrective action is taken, at which time the original test frequency shall be resumed. In any case, any abnormality or erratic action shall be reported.

Contrary to the above, the stroke time for valve CV-3-4658A, tested on August 7, 1988, and valve CV-4-2907, tested January 7, 1988, increased by more than the Section XI limits of 50 percent and the test frequency was not increased nor was adequate corrective action taken.

This is a Severity Level IV violation (Supplement I).

Pursuant to the provisions of 10 CFR 2.201, Florida Power Company is hereby required to submit a written statement or explanation to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555, with a copy to the Regional Administrator, Region II, and a copy to the NRC Resident Inspector, Turkey Point, within 30 days of the date of the letter transmitting this Notice. This reply should be clearly marked as a "Reply to a Notice of Violation" and should include [for each violation]: (1) admission or denial of the violation, (2) the reason for the violation if admitted, (3) the corrective steps which have been taken and the results achieved, (4) the corrective steps which will be taken to avoid further violations, and (5) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending the response time. If an adequate reply is not received within the time specified in this Notice, an order may be issued to show cause why the license should not be modified, suspended, or revoked or why such other action as may be proper should not be taken.

FOR THE NUCLEAR REGULATORY COMMISSION

Caudle A. Julian

Caudle A. Julian, Chief
Engineering Branch
Division of Reactor Safety

Dated at Atlanta, Georgia
this 11 day of April 1989

