



UNITED STATES  
 NUCLEAR REGULATORY COMMISSION  
 WASHINGTON, D. C. 20555

May 14, 1985

Docket Nos. 50-250  
 and 50-251

Mr. J. W. Williams, Jr., Group Vice President  
 Nuclear Energy Department  
 Florida Power and Light Company  
 Post Office Box 14000  
 Juno Beach, Florida 33408

Dear Mr. Williams:

SUBJECT: SUPPLEMENT TO THE SAFETY EVALUATION IN SUPPORT OF AMENDMENT  
 NOS. 99 AND 93 FOR THE TURKEY POINT PLANT

On December 23, 1983, the Commission issued Amendment No. 99 to Facility Operating License No. DPR-31 and Amendment No. 93 to Facility Operating License No. DPR-41 for the Turkey Point Plant Units 3 and 4, respectively. The amendments changed the Technical Specifications to support the integrated program for vessel flux reduction and to take credit for operation with the new steam generators. The amendments are the subject of a current proceeding, ASLB No. 84-504-07 LA.

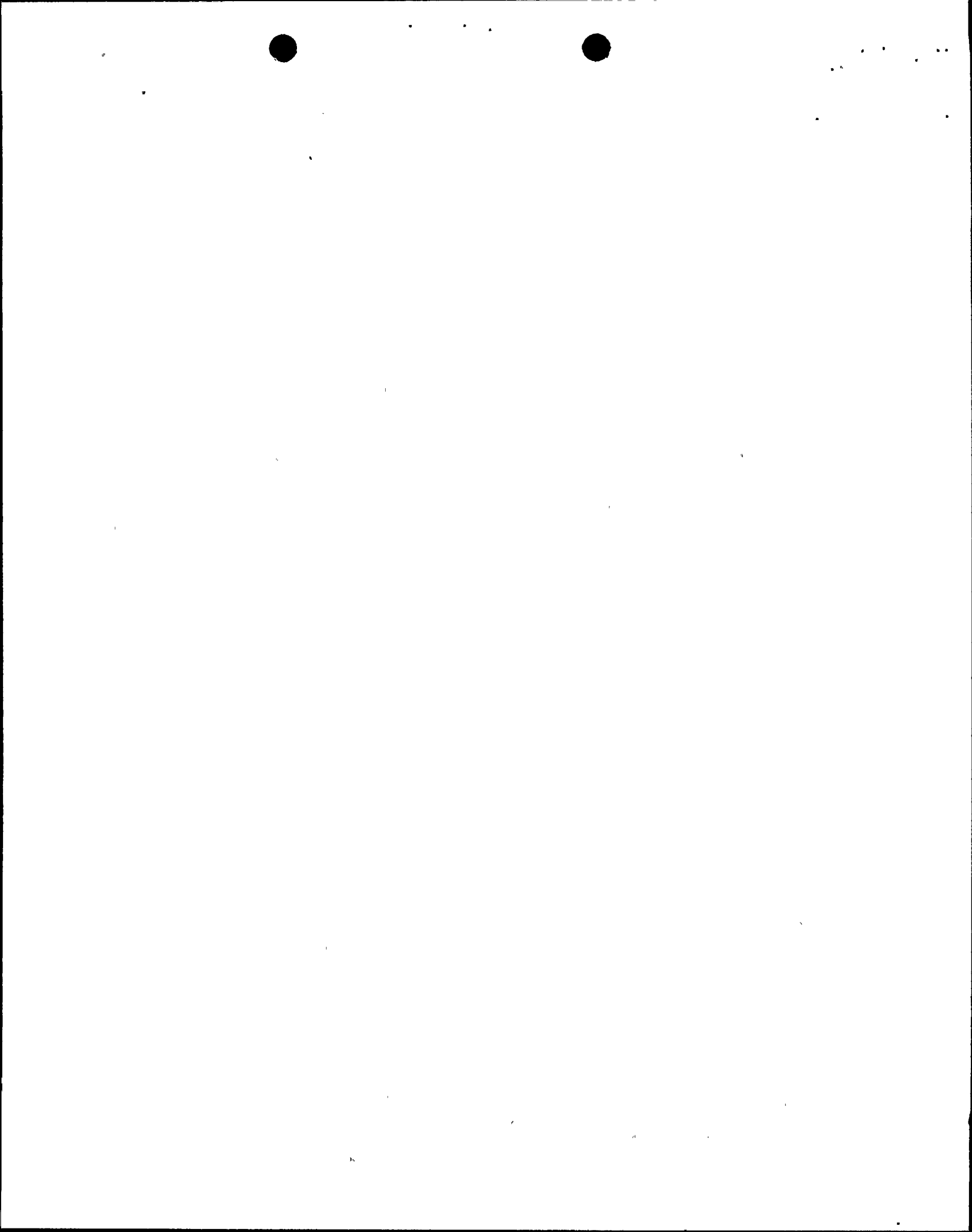
The Board was notified by letter dated March 18, 1985 from your attorney, Mr. Michael Bauser, that it was necessary to revise the data transfer procedure between the WREFLOOD and BART codes which are part of the Westinghouse Emergency Core Cooling System (ECCS) evaluation model used to demonstrate compliance with 10 CFR 50.46 and Appendix K to 10 CFR 50. It was further stated that the new procedure resulted in an increase in calculated Peak Clad Temperature (PCT) but would be well below the 2200°F limit established in 10 CFR 50.46.

By letter dated March 22, 1985 from E. P. Rahe (Westinghouse) to D. G. Eisenhut (NRC), the NRC staff was informed of the problem with the data transfer procedure between the WREFLOOD and BART codes. Details which led to the identification of the problem, corrective actions taken and a reanalysis for the Turkey Point Plant were provided.

You confirmed the results of the reanalysis by letter dated April 16, 1985 from J. W. Williams (FPL) to S. A. Varga (NRC). The new results show an increase in the calculated PCT of 79°F to 2051°F from the original calculated PCT of 1972°F. The new value is well below the 2200°F limit established in 10 CFR 50.46. In our initial Safety Evaluation (SE) we indicated that the results of an analysis using the previously approved ECCS evaluation model using the FLECHT correlation in lieu of the BART code also resulted in a PCT less than the 2200°F limit.

We have concluded in the enclosed SSE that the ECCS evaluation model using the BART code meets the requirements of 10 CFR 50, Appendix K, and the results of the reanalysis are acceptable. Our initial conclusions have remained valid based on the results of the analyses using the FLECHT correlation and all other conclusions regarding BART and the results of the large break LOCA analysis (Section 4.2 of our SE) remain valid.

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Mr. Williams

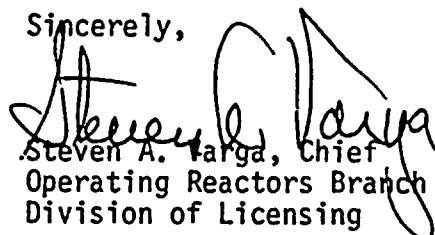
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As the result of our SSE, we have revised the PCT value of 1972°F to 2051°F in Sections 4.2 and 7 and the value of the reduction in PCT has been revised from 160°F to 81°F in Section 7 of the SE, dated December 23, 1983, issued in support of the subject amendments. The revised SE dated May 14, 1985 is included as Enclosure 2 and replaces our initial SE.

The acceptability of the new methodology for data transfer procedure between the WREFLOOD and BART codes is reflected in an SSE on the BART computer code methodology. Reference 14 of Enclosure 2, which references the initial SE for the BART code has been revised to also include a reference to the SSE for the BART code.

Sincerely,



Steven A. Farga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

1. SSE Related to Amendment 99 to DPR-31 and Amendment No. 93 to DPR-41 for Turkey Point Units 3 & 4
2. Safety Evaluation, Rev. 1 dated May 14, 1985

cc w/enclosures:  
See next page

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Florida Power and Light Company

Turkey Point Plant

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