February 26, 1985

Docket Nos. 50-250
and 50-251

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Mr. J. W. Williams, Jr., Vice President
Nuclear Energy Department
Florida Power and Light Company
Post Office Box 14000
Juno Beach, Florida 33408

Dear Mr. Williams:

Reference: Tac Nos. 56875 and 56876

SUBJECT: REACTOR PLANT SURVEILLANCE MATERIAL PROGRAM - PROPOSED LICENSE AMENDMENT, TURKEY POINT PLANT UNITS 3 AND 4

By letter dated February 8, 1985, you requested license amendments to combine the reactor materials surveillance programs of Unit 3 and Unit 4 into a single integrated program which will conform to the requirements of 10 CFR 50 Appendices G and H. We are in the process of reviewing the Safety Evaluation (SE) provided with your submittal in support of your proposed license amendments.

We have enclosed a request for additional information (RAI). This information is needed to confirm and clarify the details provided in your submittal. We request a prompt reply to the RAI in order to meet our current review schedule and allow for implementation of the proposed program during the upcoming Unit 3 outage. We are available to discuss the RAI and provide any clarification necessary.

The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

/s/DMcDonald for S. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

ORB#1:DL
DMcDonald/ts
02/26/85

BC-ORB#1:DL
SVarga
02/27/85

J. W. Williams, Jr.
Florida Power and Light Company

Turkey Point Plants
Units 3 and 4

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ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION

TURKEY POINT PLANT UNITS 3 & 4

INTEGRATED REACTOR SURVEILLANCE PROGRAM

1) Are the materials and designs for the cores, thermal shield, core barrels and vessels the same for each unit?

2) Is the figure in Attachment 3 representative for units 3 and 4?

3) What are the materials (base metals and weld metals) in the beltline of each vessel? Indicate the material specification, heat nos. (flux and weld wire), amounts of Cu, Ni, & P and initial RT_{NDT} for each matl. How were initial RT_{NDT}, Cu, Ni & P determined?

4) How was it determined that SA 1101 was the limiting matl.? What is the projected end of life RT_{NDT} for the matl. and how does it compare with the values for other materials in the beltline?

5) Describe the excore dosimetry program and how it will be used to determine the peak neutron fluence for each vessel? Has bench marking been completed?

6) Are the neutron specturm and neutron flux the same for each vessel at the peak fluence locations? Are the neutron spectrum and neutron flux the same for capsules T, V and X in each vessel? How will the dosimetry from one vessel be used to estimate the neutron fluence & spectrum for each vessel?

7) What is the end of life peak neutron fluence at the ¼T location? How was the value and the value for end of life peak neutron fluence at the inside surface determined?

8) What is the projected neutron fluence to be received by each capsule at the time of its withdrawal? Does the withdrawal sch. comply with ASTM E-185-82? If it does not, state the reason why the requested schedule is more appropriate.

Where numbers, calculation methods and programs are reported, provide references and letter identification for submittal of the information to the NRC. Were these documents approved by the NRC?