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ACCESSION NBR:8804260303 DOC.DATE: 88/04/15 NOTARIZED: NO DOCKET #
FACIL:50-250 Turkey Point Plant, Unit 3, Florida Power and Light C 05000250
AUTH.NAME AUTHOR AFFILIATION
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CONWAY,W.F. Florida Power & Light Co.
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 88-004-00:on 880318,auxiliary feedwater initiation on low steam generator level.

W/8 ltr.

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TITLE: 50.73 Licensee Event Report (LER), Incident Rpt, etc.

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RIDS/ADDS

NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Turkey Point Unit 3	0 5 0 0 0 2 5 0	1 OF 0 3

TITLE (4) Auxiliary Feedwater Initiation on Low Steam Generator Level Due to Inadequate Monitoring of Steam Generator Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 3	1 8	8 8	8 8	0 0 4	0 0	0 4	1 5	8 8	Turkey Point Unit 4	0 5 0 0 0 2 5 1
										0 5 0 0 0

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 1 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Gabe Salamon, Compliance Engineer	3 0 5 2 4 6 - 6 5 6 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 18, 1988, Unit 3 was in mode 3, with the Reactor Coolant System being cooled using the Steam Generators (SG). SG levels were being controlled manually using the Bypass Feedwater Control Valves (FCV). The Reactor Control Operator (RCO) was attempting to maintain level within the approximate 50% to 70% range on the narrow range SG level indicator. At approximately 1050 the 3B SG level reached 68% and the RCO decreased the flow. By about 1120 the SG level had dropped to the Lo SG level setpoint of 35%, and at 1129 the SG level reached the Lo-Lo SG level setpoint of 15%. Upon reaching the Lo-Lo setpoint on 2 out of 3 channels, the 3 Auxiliary Feedwater Pumps (AFW) received a start signal and delivered water to all 3 SG's. Upon the SG 3B water level returning to the operating range, the AFW pumps were secured and SG level continued to be controlled manually using the Bypass FCVs. The cause of the AFW actuation was personnel error in that the RCO failed to adequately monitor the SG level. The RCO's attention was focused on performing an operability test of a NIS channel, and because of this distraction, he failed to take corrective actions to return the SG level to the operating range upon the Lo level alarm annunciating at the 35% level, prior to the automatic AFW actuation. The operator was counseled concerning awareness of his actions and the seriousness of this error.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Turkey Point Unit 3	0 5 0 0 0 2 5 0	8 8	— 0 0 4	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

EVENT

On March 18, 1988, Unit 3 was in mode 3, with the Reactor Coolant System (EIIS:AB) being cooled using the Steam Generators (SG) (EIIS:SB) and the atmospheric dump valves. SG levels were being controlled manually using the Bypass Feedwater Control Valves (FCV) (EIIS:SJ). Blowdown to control SG chemistry was in progress at the time. The Reactor Control Operator (RCO) was attempting to maintain level within the approximate 50% to 70% range on the narrow range SG level indicator. At approximately 0950, the SG 3B level was at 67%, and by about 1025, the level had fallen to about 48%. The RCO increased the flow to the SG such that by 1050, the level had increased to 68%. The RCO then decreased the flow and at about 1120 the SG level had dropped to the Lo SG level setpoint of 35%, and by 1129 the SG level had dropped to the Lo-Lo SG level setpoint of 15%. Upon reaching the Lo-Lo setpoint on 2 out of 3 channels, the 3 Auxiliary Feedwater Pumps (AFW) (EIIS:BA) received a start signal, started, and delivered water to all 3 SG's. Upon the SG 3B water level returning to the normal range, the AFW pumps were secured and SG level continued to be controlled manually using the Bypass FCVs.

CAUSE OF EVENT

The cause of the AFW actuation was personnel error in that the RCO failed to adequately monitor the SG level. The RCO was continuously involved in periodic testing starting at 1030. At the time of the event, the RCO's attention was focused on performing an operability test of a Nuclear Instrumentation System (NIS) (EIIS:IG) channel, and because of this distraction, he failed to take corrective actions to return the SG level to the normal range upon the Lo level alarm annunciating at the 35% level, prior to the automatic AFW actuation.

ANALYSIS OF EVENT

At the time of the automatic AFW actuation, the main feedwater and standby feedpumps were operable. Upon the 3B SG water level reaching the 15% level on the narrow range SG water level indicator, all 3 AFW pumps started and delivered water to the Unit 3 SGs. Based on the above, the health and safety of the public were not affected.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT *(If more space is required, use additional NRC Form 366A's)* (17)

CORRECTIVE ACTIONS

1) The operator was counseled concerning awareness of his actions and the seriousness of this error.

2) An entry was made in the Short Term Instructions and Shift Information Book, to be discussed with the oncoming shifts, requiring increased operator awareness of the SG status whenever the SG's are fed manually, and stating that any testing on the unit should be performed by the third licensed operator, when possible.

3) This event will be reviewed by the Training Department to determine any additional training requirements.

ADDITIONAL DETAILS:

Similar occurrences: LER 251-86-17.

P. O. BOX 14000, JUNO BEACH, FL 33408-0420

FPL

APRIL 15 1988

L-88-179
10 CFR 50.73

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Re: Turkey Point Unit 3
Docket No. 50-250
Reportable Event: 88-04
Date of Event: March 18, 1988
Auxiliary Feedwater Initiation on
Low Steam Generator Level Due to
Inadequate Monitoring of Steam Generator Level

The attached Licensee Event Report (LER) is being submitted pursuant to the requirements of 10 CFR 50.73 to provide notification of the subject event.

Very truly yours,

W. F. Conway
Acting Group Vice President
Nuclear Energy

WFC/SDF/gp

Attachment

cc: Dr. J. Nelson Grace, Regional Administrator,
Region II, USNRC
Senior Resident Inspector, USNRC, Turkey Point Plant

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SDF3.LER