

ENCLOSURE 1

NOTICE OF VIOLATION

Florida Power and Light Company  
Turkey Point Units 3 and 4

Docket Nos. 50-250 and 50-251  
License Nos. DPR-31 and DPR-41

During the Nuclear Regulatory Commission (NRC) inspection conducted on August 24 - September 21, 1987, violations of NRC requirements were identified. The violations involved: the failure to properly implement maintenance procedures for safeguards circuitry fuse control, the failure to implement a required post-maintenance functional test and the failure to install a correctly sized check valve. In accordance with "General Statement of Policy and Procedure for NRC Enforcement Action," 10 CFR Part 2, Appendix C (1986), the violations are listed below.

- A. Technical Specification 6.8.1 requires that written procedures and administrative policies be established, implemented and maintained that meet or exceed the requirements and recommendations of sections 5.1 and 5.3 of ANSI N18.7-1972 and Appendix A of USNRC Regulatory Guide 1.33.

ANSI N18.7-1972, section 5.1, specifies in part that maintenance affecting the performance of safety-related equipment shall be performed in accordance with written procedures or documented instructions appropriate to the circumstances.

Maintenance Procedures (MPs) 0707.16 through 0707.20, dealing with the repair and calibration of reactor safeguards protection circuitry, specify the size and type of fuses to be installed in the respective circuits.

Contrary to the above, fuse inspections performed during the week of September 14, 1987, revealed that numerous fuses installed in the reactor safeguards protection circuitry did not match the procedurally specified fuses in either size or type.

This is a Severity Level IV violation (Supplement I).

- B. 10 CFR 50, Appendix B, Criterion V, as implemented by Florida Power and Light Topical Quality Assurance Report FPLTQAR 1-76A, Revision 10, and TQR 5.0, Revision 6, entitled Instructions, Procedures and Drawings, requires in part, that activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, of a type appropriate to the circumstances and shall be accomplished in accordance with these instructions, procedures and drawings.



1. Non-Conformance Report (NCR) C-655-87, specifies that an electrical wire splice for steam flow transmitter FT-3-475 be repaired in accordance with procedure 5610-E-1593/87-093, Revision 0, entitled Acceptance Criteria and Installation Details for Raychem Splices. Section 8.0 of the procedure specifies, in part, that post maintenance functional tests shall be performed on instrument loops receiving wire splice repairs or replacements.

Contrary to the above, subsequent to the repair of wire splice 3IFSGS/T3I22-FT475/1, instrument loop FT-3-475 did not receive a functional test. Subsequently, the instrument loop was determined to be inoperable due to a wiring discrepancy.

2. Plant Change Modification (PCM) 85-175, entitled Unit 3 Auxiliary Feedwater Nitrogen Station Additions and Relocation, specifies in drawing 5610-J-558, Revision 2, that a train check valve requiring 0.33 pounds per square inch (psi) differential to seat the disc be installed adjacent to pressure regulator 1706.

Contrary to the above, in September 1987, subsequent to system installation, inspection and testing, a check valve requiring 10.0 psi differential to seat the disc was found to be installed adjacent to pressure regulator 1706.

This is a Severity Level IV violation (Supplement I) and is applicable to Unit 3 only.

Pursuant to the provisions of 10 CFR 2.201, Florida Power and Light Company is hereby required to submit to this Office within 30 days of the date of the letter transmitting this Notice a written statement or explanation in reply including (for each violation): (1) admission or denial of the violation, (2) the reason for the violation if admitted, (3) the corrective steps which have been taken and the results achieved, (4) the corrective steps which will be taken to avoid further violations, and (5) the date when full compliance will be achieved. Where good cause is shown, consideration will be given to extending the response time.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Luis A. Reyes, Director  
Division of Reactor Projects

Dated at Atlanta, Georgia  
this 26th day of October 1987

