### REGULATORY PFORMATION DISTRIBUTION SYSTEM (RIDS)

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FACIL:50-250 Turkey Point Plant, Unit 3, Florida Power and Light C 05000250
50-251 Turkey Point Plant, Unit 4, Florida Power and Light C 05000251

AUTHOR AFFILIATION WILLIAMS, J.W. Florida Power & Light Co.

RECIP.NAME RECIPIENT AFFILIATION

VARGAS, S.A. Operating Reactors Branch 1

SUBJECT: Forwards nevised pages to inservice test program for pumps & valves, in response to NRC 850520 request for reverse reflect test requirements for MOV=750 & MOV=751.

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NOTES:

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OL:07/19/72

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Office of Nuclear Reactor Regulation

Attention: Mr. S. A. Varga, Chief

Operating Reactors Branch #1

Division of Licensing

U. S. Nuclear Regulatory Commission

Washington, D. C. 20555

Dear Mr. Varga:

Re: Turkey Point Units 3 & 4

Docket Nos. 50-250 and 50-251 Revision to the Inservice Test Program for Pumps and Valves

In a May 20, 1985 letter (L-85-204) to you requesting a revision to the Inservice Test Program for Pumps and Valves for Turkey Point Units 3 and 4, we stated that we would submit revised program pages reflecting the test requirements for MOV-750 and MOV-751 found acceptable in your letter and safety evaluation dated May 21, 1985.

Attached are the revised program pages. They include the relief requests for deletion of Appendix J testing, a revised Table IA (List of Containment Isolation Valves Test to Appendix J 10 CFR 50 Requirements) and a revised Table II (Reactor Coolant System Pressure Isolation Valves). These pages replace the same numbered pages in FPL's March 30, 1984 submittal.

If you have any questions, please call us.

Williams }

Very truly yours,

J. W. Williams, Jr. Group Vice President

Nuclear Energy 😼

JWW/TCG/gp

Attachments

cc: Harold F. Reis, Esq.

Dr. J. Nelson Grace, NRC Region II

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#### Turkey Point Plant No. 3 & 4

#### RELIEF REQUEST BASIS

SYSTEM: Auxiliary Coolant, Residual Heat Removal

3. Valve: MOV-\*-751 and MOV-\*-750

Category: A Class: 1

Function: Provides a flow path from the Reactor Coolant

System to the Residual Heat Removal System

for removal of decay heat from the reactor core.

Test Requirement: IWV-3410

Basis for Relief: These valves are provided with interlocks

to prevent opening these valves when:
(1) RCS pressure is greater than 465 psig

(2) Bither MOV-\*-862A, MOV-\*-862B, MOV-\*-863A

or MOV-\*-863B of the associated unit is open.

Alternate Testing: These valves will be tested during cold

shutdowns.

Test Requirement: IWV-3420

Basis for Relief: These valves are not required to be tested

to 10 CFR Part 50, Appendix J, Type C tests.

These valves will be tested as pressure

isolation valves.

Alternate Testing: These valves will be tested as Reactor Coolant

System pressure isolation valves using methods, procedures, and acceptance criteria in Table II

and Plant Technical Specification 4.17.

#### GENERIC RELIEF REQUEST BASIS

SYSTEM: Vai

Various

2. Valve:

CV-\*-519B

CV-\*-522A HV-\*-3 CV-\*-522B . CV-\*-4659A

CV-\*-522C

HV-\*-1
POV-\*-2603

.CV-\*-4668A

MOV-\*-860A

POV-\*-2601 \*-10-582

MOV=\*-860B

MOV-\*-872

Category:

Class:

A

Function:

These valves provide for containment

isolation

Test Requirement:

IWV-3423 and IWV-3424

Basis for Relief:

The containment isolation valves identified above are tested by pressurizing the piping or ducting between two or more valves installed in the associated containment penetration. This will result in performing the CODE Category A valve seat leakage test in a reverse direction from that specified in in IWV-3423; on one or more of the valves installed in the associated containment penetration.

Alternate Testing:

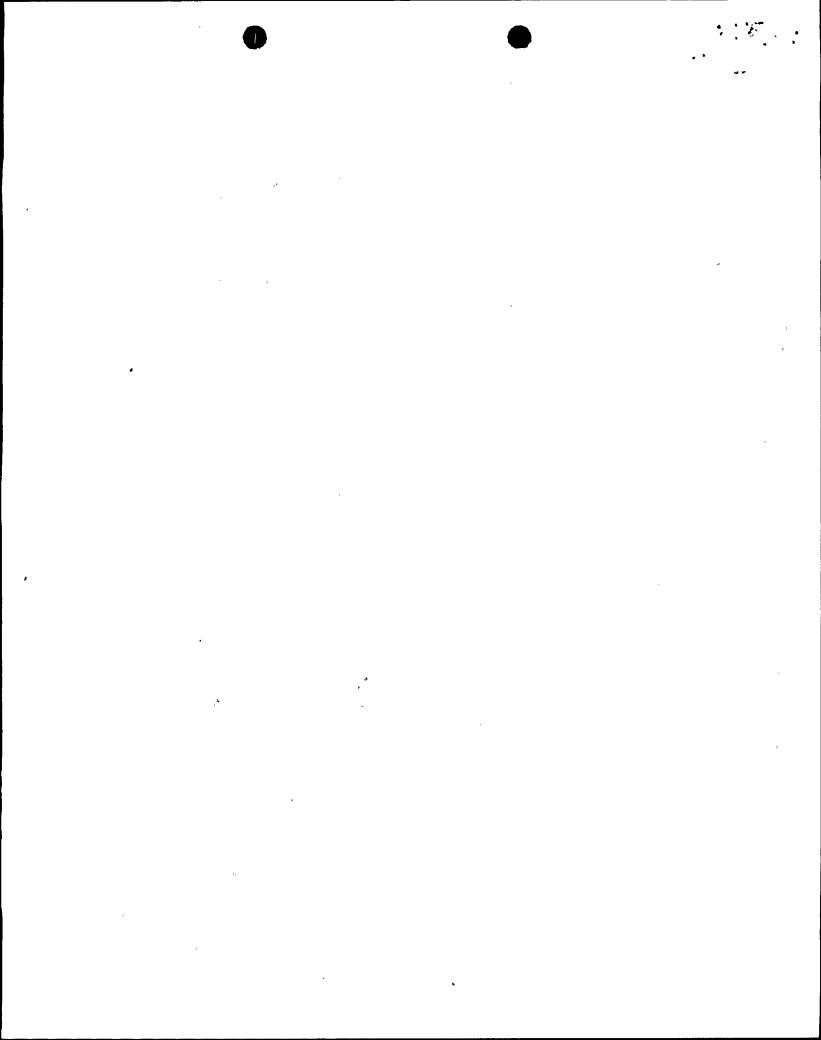
Continue to perform the CODE Category A valve seat leakage test by pressurizing the piping or ducting between two or more valves installed in the associated containment penetration.

Nothing in Section XI of the ASME Boiler and Pressure Vessel Code shall be construed as superseding the requirements of Appendix J, 10CFR50 or the Plant Technical Specifications.

Turkey Point Unit Nos. 3 & 4

TABLE IA - LIST OF CONTAINMENT ISOLATION VALVES TESTED TO APPENDIX J; 10CFR50 REQUIREMENTS .

Containment Penetration Number	Service	Valve Number(s)	Leakage Rate CC/Min
·			
5	Gas Analyzer	CV-*-516	1 000
	Sample PRT	SV-*-6385	1 000
6	Nitrogen Supply	*-518	2 500
	to PRT	*-519	2 500
7.	PW Supply to PRT and RCP Standpipes	CV-*-519A CV-*-519B CV-*-522A CV-*-522B CV-*-522C	2 000 2 000 2 000 2 000 2 000
8	Sample PRZ	CV-*-951	1 000
	Steam Space	CV-*-956A	1 000
9 ·	Sample PRZ	CV-*-953	1 000
	Liquid Space	CV-*-956B	1 000
10	Vent and	PCV-*-1014	2 000
	No Supply	CV-*-4658B	2 000
	for RCDT	CV-*-4658A	2 000
11	Alt. Low Head Safety Injection	MOV-*-872	8 000
14	Letdown	CV-*-200A	3 000
	from Reactor	CV-*-200B	3 000
	Coolant	CV-*-200C	3 000
	System	CV-*-204	3 000
15	Charging to	HCV-*-121	~ 2 000
	Reactor Coolant	*-333	2 000
	System	*-3120	4 000



# TABLE II

## REACTOR COOLANT SYSTEM PRESSURE ISOLATION VALVES

<del></del>			Maximum (a) (b)
SYSTEM	VALVE N	10.	Allowable Leagage-gpm
High-Head Safety Injection	Unit 3	Init 4.	•
Loop A, hot leg cold leg cold leg	3-875A 4	I-874A I-875A I-873A	5.0 5.0 5.0
Loop B, hot leg cold leg cold leg	3-875B	l-874B l-875B l-873B	5.0 5.0 5.0
Loop C, cold leg cold leg		l-875C l-873C	5.0 5.0
Residual Heat Removal	L		,
Loop A, cold leg		I-876 <b>a</b> I-8762	5.0 5.0
Loop B, cold leg		-876B -876D	5.0 5.0
Loop C, cold leg	3-876C 4 3-876E	1-876C	5.0 5.0
Loop A, hot leg		10V-4-750 10V-4-751	5.0 5.0
Loop C, hot leg	MOV-3-750 MOV-3-751		5.0 5.0