ATTACHMENT 1

The proposed amendment is described below and shown on the accompanying Technical Specifications, Pages:

Page 3.1-5 and Table 3.5-3 - The operating conditions for purge isolation have been added.

Table 3.5-4 (Contd.)

The setpoints for Containment Radioactivity - High has been added.

Table 4.1-1 (Sheet 4)

The surveillance requirements for the high containment radioactivity channels have been modified.

Table 4.1-1 (Sheet 3)

The surveillance requirements for the Turbine Trip have been added (Auto-Stop Oil Pressure switches)

Page 6-21

The requirement for prompt reporting of pressurizer PORV's and safety valves failure has been added.

Page B2.3-3

The reactor trip upon turbine trip concept has been added.

e. After shutdown, corrective action shall be taken before operation is resumed.

F. Above 2% of rated power, two leak detection systems of different principles shall be operable, one of which is sensitive to radioactivity. The latter may be out of service for 48 hours provided 1) Two other systems are operable and 2) containment purge valves are maintained closed.

g. Reactor Coolant System leakage shall be limited to 1 gpm total primary-to-secondary leakage through all steam generators not isolated from the Reactor Coolant System and 500 gallons per day through any one steam generator not isolated from the Reactor Coolant system.

4. MAXIMUM REACTOR COOLANT ACTIVITY

The total specific activity of the reactor coolant due to nuclides with half-lives of more than 30 minutes, excluding tritium, shall not exceed 135/E* μCi/cc whenever the reactor is critical or average reactor coolant temperature is greater than 500F. The concentration of radioiodine in the reactor coolant shall be limited to 1.0 microcurie/gram during normal operation and to 30 microcuries/gram during power transients.

If the limit above is not satisfied, the reactor shall be shutdown and cooled to 500°F or less within 6 hours.

*E is the average of beta and gamma energy (Mev) per disintergration of the specific activity.

TABLE 3.5-3

INSTRUMENT OPERATING CONDITIONS FOR ISOLATION FUNCTIONS

NO.	FUNCTIONAL UNIT	1 MIN. OPERABLE CHANNELS	2 MIN. DEGREE OF REDUN-DANCY	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET
1.	CONTAINMENT ISOLATION			
1.1	Manual	2	*	Cold Shutdown
1.2	Safety Injection	See Item No. 1 of Table 3.5-2		Cold Shutdown
1.3	High Containment Pressure	See Item 2.1 of Table 3.5-2		Cold Shutdown
2.	STEAM LINE ISOLATION			
2.1	High Steam Flow in 2/3 Lines and 2/3 Low T_{avg} or 2/3 Low Steam Pressure	See Item 1.5 in Table 3.5-2		Cold Shutdown
2.2	High Containment Pressure	See Item No. 2.1 of Table 3.5-2		Cold Shutdown
2.3	Manual	1/line		Hot Shutdown
3.	FEEDWATER LINE ISOLATION			
3.1	Safety Injection	See Item No. 1 of Table 3.5-2		Cold Shutdown
4.	PURGE ISOLATION			
4.1	Containment High Radioactivity	1 **	0	***

* Must actuate two push buttons simultaneously

** Either particulate or gaseous

*** With Tavg at or above 200°F, and with less than the minimum operable channels, operations may continue provided the containment purge valves are maintained closed.

TABLE 3.5-4 (Contd.)

NO.	FUNCTIONAL UNIT	CHANNEL ACTION	SET POINT
10.	Containment Radioactivity - High	Close Purge valves**	Particulate (R-11) $\leq 6.1 \times 10^5$ CPM** Gaseous (R-12) See Note 1 **
11.	Turbine Auto-stop Oil Pressure - Low	1) Turbine Trip 2) Reactor Trip above P-7	$<$45 psig
12.	Turbine Stop Valves	1) Turbine Trip 2) Reactor Trip above P-7	Both valves closed

NOTE 1 R-12 Setpoint = $\frac{(3.2 \times 10^4)}{(F)}$ CPM, where F = $\frac{\text{Actual Purge Flow}}{\text{Design Purge Flow (35,000 CFM)}}$

Set point may vary according to current plant conditions provided that release rate does not exceed allowable limits specified on TS.3.9.2.b.

** With Tavg at or above 200° F, these setpoints could be higher provided that the purge valves are maintained closed. A purge may be initiated if either setpoint is higher than the allowable value, if the other setpoint is at or below its allowable value.

* Using moveable in-core detector system.
** Frequency only
*** Effluent monitors only and R-11 & R-12. Calibration for effluent monitors shall be as specified in 3.9.

S	-	Each Shift
D	-	Daily
W	-	Weekly
B/W	-	Every Two Weeks
M	-	Monthly
Q	-	Quarterly
P	-	Prior to each startup if not done previous week
R	-	Each Refueling Shutdown
A	-	Annually
N.A.	-	Not appliable
†	-	N.A. during cold or refueling shutdowns. The specified tests, however, shall be performed within one surveillance interval prior to startup.
††	-	N.A. during cold or refueling shutdowns. The specified tests, however, shall be performed within one surveillance interval prior to heatup above 200F.

TABLE 4.1-1 SHEET 3

	Channel Description	Check	Calibrate	Test	Remarks
23.	Environmental Radiological Monitors	N.A.	A(1)	M(1)	(1) Flow
24.	Logic Channels	N.A.	N.A.	M[†]	
25.	Emer. Portable Survey Instruments	N.A.	A	M	
26.	Seismograph	N.A.	N.A.	Q	Make trace. Test battery (change semi-annually)
27.	Auxiliary Feedwater Flow Rate	M[†]	R	N.A.	
28.	RCS Subcooling Margin Monitor	M[†]	R	N.A.	
29.	PORV Position Indicator (Primary Detector)	M[†]	N.A	R	Check consists of monitoring indicated position and verifying by observation of related parameters
30.	PORV Block Valve Position Indicator	M[†]	N.A	R	
31.	Safety Valve Position Indicator	M[†]	R	N.A.	
32.	Loss of Voltage (both 4kv busses)	N.A.	N.A.	R	For AFW actuation at power only
33.	Trip of both Main Feedwater Pump Breakers	N.A.	N.A.	R	For AFW actuation at power only
34.	Turbine Trip (Auto-Stop Oil Pressure Switches)	N.A.	R.	N.A.	

NOTE: This item is intended to provide for reporting of potentially generic problems.

(10) Failure of the pressurizer PORVs or safety valves.

b. Thirty Day Written Reports The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

(1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.

(2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

(3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to: cause reduction of degree of redundancy provided in reactor protection systems or enginered safety feature systems.

(4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

Reactor Trip Interlocks

Specified reactor trips are by passed at low power where they are not required for protection and would otherwise interfere with normal operation. The prescribed set points above which these trips are made functional assures their availability in the power range where needed.

An automatic reactor trip will occur if any pump is lost above 55% power which will prevent the minimum value of the DNBR from going below 1.30 during normal and anticipated transient operations when only two loops are in service, (9) and the over temperature ΔT trip setpoint is adjusted to the value specified for three loop operation.

A turbine trip initiates a reactor trip. On decreasing power, the turbine trip is automatically blocked by P-7; and on increasing power reinstated automatically by P-7.

Reset of reactor trip interlocks will be done under strict administrative control.

REFERENCES

(1) FSAR 14.1.1

(2) FSAR 14.1.2

(3) FSAR 14.1

(4) FSAR 7.2, 7.3

(5) FSAR 3.2.1

(6) FSAR 14.3.1

(7) FSAR 14 (Page 14-3) and 14.1.9

(8) FSAR 14.1.11

(9) FSAR 14.1.9

(10) WCAP-8074

STATE OF FLORIDA)
) ss.
COUNTY OF DADE)

J. W. Williams, Jr., being first duly sworn, deposes and says:

That he is Vice President of Florida Power & Light Company, the herein;

That he has executed the foregoing document; that the statements made in this said document are true and correct to the best of his knowledge, information, and belief, and that he is authorized to execute the document on behalf of said

J. W. Williams, Jr.

Subscribed and sworn to before me this 29th day of December, 1982

Lourdes Jordan
NOTARY PUBLIC, in and for the County of Dade, State of Florida

NOTARY PUBLIC STATE OF FLORIDA AT LARGE
MY COMMISSION EXPIRES DEC 8 1984
BONDED THRU GENERAL INS. UNDERWRITERS

My commission expires: