



Entergy Operations, Inc.
P.O. Box 756
Port Gibson, Mississippi 39150

Eric L. Larson
Site Vice President
Grand Gulf Nuclear Station
Tel: 601-437-7500

GNRO-2017/00079

December 7, 2017

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Subject: Supplement to License Amendment Request for One Cycle Extension Of
Appendix J Type A Integrated Leakage Rate Test and Drywell Bypass Test
Interval
Grand Gulf Nuclear Station, Unit No. 1
Docket No. 50-416
License No. NPF-29

REFERENCE: GNRO-2016/00062, License Amendment Request for One Cycle
Extension Of Appendix J Type A Integrated Leakage Rate Test and
Drywell Bypass Test Interval, dated December 29, 2016

Dear Sir or Madam:

This letter provides a supplemental change to the previously provided Technical Specification Markup, pages 3.6.53 and 5.0-16. Please replace the previously submitted pages with those provided in the attachment.

This letter contains no Regulatory Commitments.

If you have any questions or require additional information, please contact Robert Peters at 601-437-2318 or Douglas Neve, Manager Regulatory Assurance at (601)-437-2103.

Sincerely,

A handwritten signature in black ink, appearing to read "EAL", followed by a horizontal line.

Eric A. Larson
Site Vice President

EAL/saw

Attachment: Revised (Clean) Technical Specification Pages

cc: NRC Senior Resident Inspector
Grand Gulf Nuclear Station
Port Gibson, MS 39150

U.S. Nuclear Regulatory Commission
ATTN: Mr. Kriss Kennedy
Region Administrator, Region IV
1600 East Lamar Blvd, Suite 400
Arlington, TX 76011-4005

Mr. John P. Boska, Project Manager
Plant Licensing Branch 1-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Mail Stop 0-8-C2
Washington, DC 20555

Thomas J. Wengert
U.S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike
Rockville, MD 20852

Attachment

to

GNRO2017-00079

Revised (CLEAN) Technical Specification Pages

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
<p>SR 3.6.5.1.1</p> <p>Verify bypass leakage is less than or equal to the bypass leakage limit.</p> <p>However, during the first unit startup following drywell bypass leak rate testing performed in accordance with this SR, the acceptance criterion is leakage \leq 10% of the bypass leakage limit.</p>	<p>24 months following 2 consecutive tests with bypass leakage greater than the bypass leakage limit until 2 consecutive tests are less than or equal to the bypass leakage limit</p> <p><u>AND</u></p> <p>48 months following a test with bypass leakage greater than the bypass leakage limit</p> <p><u>AND</u></p> <p>-----NOTE----- SR 3.0.2 is not applicable for extensions > 12 months. -----</p> <p>120 months, except that the next drywell bypass leak rate test performed after the October 19, 2008 test shall be performed no later than the plant restart after the End of Cycle 22 Refueling Outage.</p>

(continued)

5.5 Programs and Manuals (continued)

5.5.11 Technical Specifications (TS) Bases Control Program

This program provides a means for processing changes to the Bases of these Technical Specifications.

- a. Changes to the Bases of the TS shall be made under appropriate administrative controls and reviews.
- b. Licensees may make changes to Bases without prior NRC approval provided the changes do not require either of the following:
 - 1. A change in the TS incorporated in the license; or
 - 2. A change to the updated FSAR or Bases that requires NRC approval pursuant to 10 CFR 50.59.
- c. The Bases Control Program shall contain provisions to ensure that the Bases are maintained consistent with the UFSAR.
- d. Proposed changes that do not meet the criteria of either Specification 5.5.11.b.1 or Specification 5.5.11.b.2 above shall be reviewed and approved by the NRC prior to implementation. Changes to the Bases implemented without prior NRC approval shall be provided to the NRC on a frequency consistent with 10 CFR 50.71(e).

5.5.12 10 CFR 50, Appendix J, Testing Program

This program establishes the leakage rate testing program of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be implemented in accordance with the Safety Evaluation issued by the Office of Nuclear Reactor Regulation dated April 26, 1995 (GNRI-95/00087) as modified by the Safety Evaluation issued for Amendment No. 135 to the Operating License, except that the next Type A test performed after the October 19, 2008 Type A test shall be performed no later than the plant restart after the End of Cycle 22 Refueling Outage. For Type B and Type C local leakage rate testing, this program shall be in accordance with the guidelines contained in NEI 94-01, Revision 3-A, "Industry Guideline for Implementing Performance-Based Option of 10 CFR Part 50, Appendix J," dated July 2012. Consistent with standard scheduling practices for Technical Specifications required surveillances, intervals for the recommended surveillance frequency for Type A testing may be extended by up to 25 percent of the test interval, not to exceed 15 months. The calculated peak containment internal pressure for the design basis loss of coolant accident, Pa, is 12.1 psig.