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SUBJECT: Submits integrated startup test program for insp & testing to be completed prior to returning unit to operation after steam generator repair program, per Amend 69° to OL, Apps A, B, C & D encl.

Repts. SEE

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P.O. BOX 529100 MIAMI, FL 33152



FLORIDA POWER & LIGHT COMPANY

December 23, 1981 L-81-535

Office of Nuclear Reactor Regulation Attention: Mr. Steven A. Varga, Chief Operating Reactors Branch No. 1 Division of Licensing U. S. Nuclear Regulatory Commission Washington, D. C. 2055

Dear Mr. Varga:

Re: Turkey Point Unit No. 3 Docket No. 50-250 <u>Integrated Startup Test Program</u>

Integrated Startup. Test Program We are submitting, for your review, our Integrated Startup Test Program for inspection and testing to be completed prior to returning Turkey Point Unit No. 3 to operation after the Steam Generator Repair Program. This letter is to meet the requirements of Amendment No. 69 to the Operating License for Turkey Point Unit No. 3 to submit the test program sixty days prior to fuel loading. The Integrated Startup Test Program is comprised of two phases: Preoperational Tests and Startup Tests. The format of the program follows the intent of Regulatory Guide 1.68, Revision 2, August, 1978, "Preoperational and Initial Startup Test Programs for Water-Cooled Power Reactors". In some cases interpretation is necessary since Turkey Point Unit No. 3 is a previously licensed PWR plant that has undergone major equipment repair and modification rather than being new construction.

The tests in the Preoperational Test phase are designed to provide assurance that components and subsystems of new, modified, and original systems function safely within established design criteria. The Preoperational Tests on a new or modified system are conducted prior to fuel loading. This test phase also allows the plant operating staff to become familiar with the operation of a new or modified system and to verify by trial use, to the extent practical, that the operating procedures are adequate. The hot functional test of the plant will be performed during this test phase.

The tests in the Startup Test phase are designed to provide assurance that systems that were previously demonstrated as functioning safely, and the new or modified systems will function to "(1) provide for safe normal operation and high tolerance for systems malfunctions and transients, (2) ensure that, in the event of errors, malfunctions, and off-normal conditions, the reactor protection systems and other design features will arrest the event or limit its consequences to defined and acceptable levels, and (3) ensure that adequate safety margins exist for events of extremely low probability or for arbitrarily postulated hypothetical events without reduction in the safety margin for the protection of public health and safety". The Startup Tests are performed after fuel loading to confirm the design basis and demonstrate that the plant will continue to operate in accordance with design.



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Office of Nuclear Reactor Regulation Page Five

#### SUMMARY

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This letter outlines the Integrated Startup Test Program we plan to implement to demonstrate our ability to safety return Turkey Point Plant Unit No. 3 to power. The Integrated Startup Test Program will provide the necessary assurance that the plant can be operated in accordance with design requirements and in a manner that will not endanger the health and safety of the public.

Very truly yours,

Jahert Ellhung Robert E. Uhrig

Vice President Advanced Systems & Technology

REU/JEM/ah

cc: J. P. O'Reilly, Regional Administrator, Region II Harold F. Reis, Esquire

#### LIST OF APPENDICES

APPENDIX A	STARTUP	NETWORK

APPENDIX B LIST OF MAJOR PREOPERATIONAL TESTS AND CHECKS

APPENDIX C LIST OF MAJOR STARTUP TESTS AND CHECKS

APPENDIX D PROCEDURE FOR CONDUCTING THE INTEGRATED STARTUP PROGRAM



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#### APPENDIX A

#### STARTUP NETWORK

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Per Criterion 1 of Appendix A to 10 CFR 50 all structures, systems, and components will be tested or demonstrated operable to levels commensurate with the importance of the safety functions. In addition, the extent of testing will vary directly with the amount of modifications done to and around the particular equipment or system. The sequence of tests will be conducted so that the safety of the plant is never totally dependent on the performance of untested structures, systems or components.

The key points of the Integrated Startup Test Program are presented below. The attached Appendices provide additional details of the program. Further information is available for the NRC site inspector at the Turkey Point Plant. These Appendices are working documents that will be revised as required within the framework of this letter.

#### ORGANIZATION

Appendix D contains the "Procedure for Conducting the Integrated Startup Test Program for Post Steam Generator Repair and Design Changes". This document defines the organizations, responsibilities, actions, and administrative controls for each phase of the program.

The management and direction of the Integrated Startup Test Program is under the direct control of Florida Power & Light Company (FPL) with principal authority assigned to the Plant Manager-Nuclear for Turkey Point Plant. The Startup Department is composed of FPL Nuclear Operations and Maintenance personnel with outside consultants as needed. The Startup Department has overall responsibility for implementation and documentation of the program.

The conduct and direction of the tests in the Preoperational Test phase are controlled by the Startup Department. The conduct and direction of the tests in the Startup Test phase are controlled by Turkey Point Operations staff with technical support from the Startup Department.

In all cases the test procedures require approval of the Plant Nuclear Safety Committee prior to implementation. Deviations to approved test procedures are documented and become part of the final test results. Administrative controls for making changes to approved procedures are provided in existing plant procedures. The acceptance criteria for all tests are approved by the Plant Nuclear Safety Committee. Design related deficiencies are resolved by the Startup Department with the assistance of the FPL project engineer and Turkey Point operations or outside consultants as deemed necessary.

#### TEST PHASES

Preoperational and Startup Test Phases are shown on the network in Appendix A and defined in more detail in Appendix D. Each phase of the Integrated Startup Test Program is composed of a series of tests as described below:

#### I. Preoperational Test Phase

The Preoperational Test Phase consists of flushes and hydrostatic and functional tests of new, modified and affected original equipment and systems with no fuel in the reactor. This phase also includes a walkdown of existing plant systems adjacent to construction work areas for possible damage. The Hot Functional Test includes heatup of the primary system to hot shutdown conditions, thermal expansion checks of





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affected systems, vibration testing of construction affected equipment, reactor coolant pump coast down time checks and primary system operational pressure test. A primary system flush and clean-up will be performed during the Hot Functional Test at the recommendation of the NSSS supplier and to meet primary system chemistry requirements. Any repairs and subsequent testing will be accomplished as necessary prior to fuel loading and startup testing. Major milestones during the Preoperational Test phase are outlined in Appendix A. Major Preoperational Tests are listed in The actual sequence of individual tests is formulated prior to Appendix B. performance of the tests considering equipment and system availability. The Startup Department analyzes the preoperational test results. The acceptable criteria for all tests are approved by the Plant Nuclear Safety Committee. In instances where performance of components of systems deviates from predicted results, further engineering evaluations, rework, and/or retesting is performed to resolve the discrepancies before the test is considered satisfactory. Assistance from the FPL project engineer and Turkey Point operations or outside consultants is solicited as Systems which have to be modified as a result of the deemed necessary. Preoperational Test are retested to verify acceptable performance. The major prefuel loading Preoperational Tests are outlined in Appendix B. Components and systems are tested and evaluated according to approved testing procedures. Preoperational Tests are performed to verify, as near as possible, the performance of the system under actual operating conditions. Where required, simulated signals or inputs are used to verify the full operating range of the system and to calibrate and align the systems and instruments at these conditions.

#### 2. Startup Test Phase

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The major testing milestones during the Startup Test phase are identified on the network (Appendix A) and discussed below. Major Startup Tests are listed in Appendix C.

#### a. Post-Fuel Loading Tests

Systems that are not used during normal plant operations, but must be in a state of readiness to perform safety functions, are tested or demonstrated operable prior to plant conditions requiring them to be available as defined in the Technical Specifications. Fuel loading will begin after all prerequisite system tests and operations are satisfactorily completed. Upon completion of fuel loading, the reactor upper internals and pressure vessel head are installed. Additional mechanical and electrical tests are performed on the rod control system, rod position indication, and in-core moveable detector system. The purpose of this segment of the Startup Test Phase is to prepare the system for nuclear operation and to establish that all design requirements necessary for operation are achieved.

#### b. Criticality and Low Power Physics Tests

On completion of integrated safeguards testing, nuclear operation of the reactor begins. These final segments of Startup Testing include criticality and low power

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physics testing. The purpose of these tests is to verify the operational characteristics of the unit and core, and to acquire data for the proper calibration of set points, and to ensure that operation is within license requirements. Appendix C includes the major Startup Tests which are performed from fuel loading to rated power. The actual sequence of tests is formulated by the Turkey Point startup and operating personnel considering test requirements and equipment availability.

Procedures are prepared to specify the sequence of tests and measurements and the conditions under which each is to be performed to ensure safety of operation and consistency of the results obtained. If significant deviations from design calculation exist, unacceptable behavior is revealed, or apparent anomalies develop, the testing would be suspended and the situation reviewed to determine whether a question of safety is involved prior to resumption of testing.

#### c. Power Operation

After the operating characteristics of the reactor and unit have been verified by low power physics testing, the plant will be brought to full rated power level in accordance with existing plant procedures.

#### d. At Power Testing

Upon reaching full rated power level, the following at power tests are performed: final steam generator carryover testing, secondary plant heat balance checks, condensate polishing chemistry performance testing, and load rejection testing with the condensate polisher in service, when the system becomes available.

#### EXTENT OF TESTING

All Unit No. 3 systems and systems common to both Unit No. 3 and Unit No. 4 are included in the Integrated Startup Test Program. The tests required for individual components within a system will be developed by the Startup Department or will be performed in accordance with existing plant procedures. The Procedure Index for existing plant procedures is included as an Attachment to Appendix D.

In areas such as Unit No. 3 containment, where extensive work has been performed, all nuclear safety related equipment and systems will be checked by visual inspection during the Preoperational Testing or Startup Testing phase. In other areas, such as the Auxiliary Building where little work has been performed, selected system walkdowns will be employed in conjunction with normal plant startup procedures to verify the operability of the equipment.

Those systems that are new or have undergone major design basis changes will undergo complete component testing and performance testing to verify design and installation.

The accelerated start up schedule may preclude completion of previously planned modifications. All components will be tested upon completion to insure system reliability prior to being placed into service. •

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APPENDIX B

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#### LIST OF MAJOR PREOPERATIONAL TESTS AND CHECKS

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#### APPENDIX B

#### LIST OF PREOPERATIONAL TESTS AND CHECKS

#### I. HYDROSTATIC TESTING

#### TITLE OF TEST OR CHECK

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I. Steam Generator Secondary Side Integrity Test

#### TEST OR CHECK OBJECTIVE

To verify the integrity of secondary side and associated piping following the installation of the new steam generator lower assemblies.

#### II. HOT FUNCTIONAL TEST

1. The hot functional test is performed in accordance with Hot Functional Test Sequencing Document.

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The Reactor Coolant System is tested using pump heat to reverify heat-up procedures and to demonstrate satisfactory performance of components and systems exposed to reactor coolant system temperature. Proper operation of instrumentation, controllers, and alarms, checked against design operation is conditions of auxiliary systems and verified. Among the setpoints demonstrations performed are:

- a. To check that water can be charged by the Chemical and Volume Control System at rated flow against normal reactor coolant pressures.
- b. To check letdown design flow rate for each operating mode.
- c. To check response of system to change in pressurizer level.
- d. To check operation of the excess letdown and seal water flow paths.
- e. To check steam generator level instrumentation response to level changes.
- f. To check thermal expansion of selected system components and piping.
- g. To perform isothermal calibration of resistance temperature detectors and incore thermocouples.
- h. To operationally check out the residual Heat Removal System.

NOTE: The sequence of tests and checks following is not necessarily in the order that they will be performed during the Hot Functional Test.

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#### LIST OF PREOPERATIONAL TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

2. Process instrumentation (Temperature pressure, level, and flow instruments)

3. Expansion and Restraint

#### 4. Pressurizer



5. Reactor Coolant Pumps and Motors

#### TEST OR CHECK OBJECTIVE

Equipment is aligned and calibrated using existing plant procedures.

During the heat-up to operating temperature, selected points on components and piping of the Reactor Coolant System are checked at various temperatures to verify unrestricted Points of interference expansion. detected during the heat-up are recorded for correction after cool down or are increasing the corrected prior to temperature.

During the hot functional testing, the pressure controlling capability of the pressurizer is demonstrated to be within the controlling band. With reactor coolant pumps operating and with full spray, the pressure-reducing capability of the pressurizer is verified. With the spray secured and all heaters energized, the pressure-increasing capability of the pressurizer is verified.

Pressurizer Power Operated Relief Valves are functionally checked at operating pressure.

The Pressurizer Safety Valves are bench tested to verify setpoints using existing plant procedures prior to installation.

As the pumps and motors are placed in operation they are checked for:

- 1. Megger and hi pot test (as applicable)
- 2. Cooling
- 3. Lubrication
- 4. Power requirements including correct power supply voltage.
- 5. Overload protection
- 6. Direction of rotation (initial start only)
- Vibration frequency and amplitude of motor shaft and motor frame

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#### APPENDIX B

#### LIST OF PREOPERATIONAL TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

6. Steam Generators

7. Chemical Tests to Establish Water Quality

- 8. Reactor Coolant Flow Test
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- 9. Chemical and Volume Control System System Tests

#### TEST OR CHECK OBJECTIVE

The proper operation of instrumentation and control system of steam generators are checked during heat-up and at The functioning of the temperature. blowdown system will be checked.

Water for Reactor Coolant System fill and makeup is analyzed for chloride content, conductivity, total suspended solids, pH, clarity, and fluorides to requirements specified by existing plant procedures. During RCS heatup and prior to exceeding 250°F, hydrazine is added to scavenge oxygen from the RCS. Prior to, at hot shutdown and during heat-up, chemical analysis is performed to verify R.C.S. within chemistry is the specifications.

At cold shutdown conditions, measurements are made of elbow tap differential pressures to make relative comparison. At hot shutdown conditions, measurement of loop elbow differential pressure drops are made. Using these data with the reactor coolant pump performance curve, the calculated flow is verified to the design flow. Flow coastdown and transients following reactor coolant pump stoppages are also determined during the hot functional testing.

Makeup and letdown operations are conducted with the Chemical and Volume Control System to check out the different modes of dilution and boration and to verify flow in the different modes. The adequacy of heat tracing to maintain the required Boric Acid concentration in solution is verified. The ability to adequately sample is demonstrated.

The ability of the emergency borate system is verified by pumping boric acid into the Reactor Coolant System.



**10.** Emergency Boration System Tests (Safety Injection System)

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#### LIST OF PREOPERATIONAL TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

11. Steam Generator Safety Valves

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12. Auxiliary Feedwater System

13. Instrument Air System

14. Leak Detection System Tests

- 15. Residual Heat Removal System Test
- 16. Reactor Coolant Pump Seal and Cooling Water Test

17. Condenser Circulating Water System



18. Steam Generator Makeup Water and Chemical Treatment

#### TEST OR CHECK OBJECTIVE

The setpoint of safety valves is verified by tests at appropriate pressure and temperature conditions in accordance with existing plant procedures. Setpoints are checked by using a pressure assist device which adds to the force due to pressure. Once the valve leaves the seated position the assist device is vented, allowing the valve to reseat immediately.

The auxiliary feedwater system is operationally checked out to verify its ability to provide water to the steam generators within the required time frame.

The instrument air system is tested to verify proper operation.

Temperature detectors in the drain lines from pressurizer safety valves and the reactor vessel head seal and their alarm functions are checked. Pressurizer relief tank level and temperature sensors are calibrated and associated alarms checked.

The residual heat removal systems capability to remove heat is demonstrated.

Prior to reactor coolant pump operation and with the system pressurized, flow to the pump seals and cooling water is set. Flow is adjusted to specified values using installed instruments. When at operating temperature and pressure, seal and cooling flows and temperatures are checked.

Prior to hot functional testing, the main circulating water system is tested to verify operability.

The makeup system to the steam generators is checked out to verify operability. The chemical treatment system is checked out when chemicals are added to the steam generators.

#### APPENDIX B

#### LIST OF PREOPERATIONAL TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

19. Component Cooling Water System Test

20. Primary Sampling System

21. Pressurizer Relief Tank

22. Containment Ventilation System Test

23. Accumulator Tests

24. Containment Spray System Tests

25. Primary System Hydrostatic Test

#### TEST OR CHECK OBJECTIVE

Component cooling flow to the various components cooled by the CCW system is adjusted, the system operationally checked out, and setpoints verified.

Operations are performed to:

- 1. Demonstrate that liquid and gas samples can be obtained from sample points.
- 2. Demonstrate that valves, instruments, and controls function properly.
- 3. Verify proper functioning of the sample cooler.

The pressurizer relief tank and associated valves and instrumentation are checked out to verify performance of design functions.

The system is operated to balance air flows and to verify the ability to maintain temperatures below maximum allowable limits, ability to cool Rx components and RCPs.

Flow through the accumulator lines is initiated to demonstrate that the check valves are free to open. Tests are also made to verify that accumulator pressure could be maintained.

Tests are performed to verify pump operating characteristics, and response to control signals.

Hydrostatic testing of the reactor coolant system will be performed at test pressures as specified by ASME standards for the system. Prior to pressurization, the system will be heated to hot shutdown conditions. The pressure is then increased to test pressure, maintained for the specified time period and inspection for leakage is made. Overpressure protection is provided during the test.

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#### APPENDIX B

#### LIST OF PREOPERATIONAL TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

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27. Incore Thermocouples and Resistance Temperature Detectors

#### TEST OR CHECK OBJECTIVE

All readout and temperature compensating equipment is checked during the calibration and isothermal corrections for the operative thermocouples are determined.

28. Control Rod Drive Mechanism and Rod Position Indication Coil Cooling System Test

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The cooling system is checked out to verify adequate air flow, proper temperatures and motor current.

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#### LIST OF MAJOR STARTUP TESTS AND CHECKS

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#### LIST OF STARTUP TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

#### TEST OR CHECK OBJECTIVE

#### I. FUEL LOAD

- I. Refueling Equipment (Hand Tools, Power Equipment and Associated Protective Interlocks)
- 2. Nuclear Instrumentation System
- 3. Process Instrumentation (Temperature Pressure, Level and Flow Instruments)
- 4. Area Radiation Monitor Tests
- 5. Reactor Component Handling System (Polar Crane)
- 6. Residual Heat Removal System Test

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7. Chemistry Tests

8. Baseline Data for Inservice Inspections

Prior to core loading, tests and checks are performed in accordance with existing plant procedures to demonstrate the operability of the fuel handling equipment and fuel transfer system.

Nuclear instruments are calibrated and source range detector response to a neutron source is checked as a primary source is loaded.

Equipment was aligned and calibrated during Preoperational Testing.

Prior to core loading, the radiation monitoring system alarms associated with core loading are checked out and the alarm setpoints verified.

Testing was performed on the polar crane during the Construction Phase of the steam generator repair.

Testing was performed on this system during Preoperational Testing.

Prior to, during and following core loading, primary sampling will be performed to verify boron concentration in the reactor coolant system is within Tech Spec limits..

Systems and components that require inspection in accordance with Section XI of the ASME Codes are examined for baseline data. Information from these inspections provides baseline data for subsequent inservice inspections.

Note: This activity will be occurring throughout the entire test program.

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#### APPENDIX C

#### LIST OF STARTUP TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

#### TEST OR CHECK OBJECTIVE

#### II. LEAK RATE TESTING

1. Containment Tests

Containment Type B and C leakage tests will be performed in accordance with Appendix J to 10 CFR 50 and approved plant procedures.

#### **III. INTEGRATED SAFEGUARDS TEST**

- **1.** Engineered Safety Features The Engineereed Safety Features logic matrices are tested to demonstrate operability, proper logic, redundancy and coincidence in accordance with Technical Specifications. 2. Emergency Power Systems The automatic starting and loading of the diesel generators emergency is demonstrated during temporary loss of off-site power during the safeguards test. 3. Containment Isolation Systems The operation of actuation systems and components used for containment 1 × 1 isolation is verified during test. 4. Auxiliary Feedwater System Automatic operation of pumps and valves is verified during test. 5. Safety Injection System Automatic operation of pumps and valves is verified during test. 6. Residual Heat Removal System Automatic operation of pumps and valves is verified during test. 7. Containment Spray System Automatic operation of pumps and valves is verified during test. 8. Emergency Containment Coolers
  - Automatic operation of coolers, valves and required water flow is verified during test.

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#### APPENDIX C

#### LIST OF STARTUP TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

9. Emergency Containment Filters

10. Accumulator Check Valves

#### TEST OR CHECK OBJECTIVE

Automatic operation of filter fan is verified during test.

Operation of check valves will be verified during test.

#### IV. LOW POWER PHYSICS TESTS

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I. Reactor Protection System

2. Nuclear Instrumentation (Excore)



3. Control Rod Systems Tests A. Rod Control System

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B. Rod Drop Tests

C. Rod Position Indication

Prior to criticality, this system is tested to demonstrate operability, proper logic, redundancy and coincidence. The protection channels are verified through tripping of the reactor trip breakers.

Prior to criticality, all channels are checked to verify high level trip functions, alarm setpoints, operation of strip chart recorders and any auxiliary equipment.

Prior to plant heatup or at hot shutdown, this system is energized and operationally checked out with mechanisms connected to each power supply. The ability of the system to step is verified, the alarm and inhibit functions tested and the system parameters adjusted to specified values.

At hot shutdown conditions, the drop times of full length rods are tested. The drop time is measured from the release of the rod until the rod enters the top of the dashpot. This time is verified to be less than the maximum value specified in the Technical Specifications.

During rod control system tests, the position indication system was aligned to provide rod movement indication. At hot shutdown conditions, individual rod positions are calibrated to within tolerances specified in existing plant procedures.

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#### APPENDIX C

#### LIST OF STARTUP TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

4. Steam Generator Blowdown System Test

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#### 5. Initial Criticality

1. Power Ascension

#### TEST OR CHECK OBJECTIVE

At hot shutdown conditions, a functional test is performed to verify blowdown flow is consistent with system design parameters. This blowdown system flow test is also performed at 70% reactor power.

The objective is to bring the reactor critical from the plant hot shutdown conditions. Prior to start of rod withdrawal, the nuclear instrumentation checked, had been aligned, and conservative reactor trip setpoints made per existing plant procedures. At preselected points in rod withdrawal, data is taken and inverse count rate plots made to enable extrapolating to the expected critical rod position. Initial criticality and low power physics tests are performed in accordance with existing plant procedures to verify plant design parameters.

#### V. FULL POWER OPERATION

Normal post refueling Technical Specification required testing will apply for power ascension. In addition the following design tests associated with modified systems will be performed:

- I. Steam generator moisture carryover tests.
- 2. Steam generator thermal and hydraulic performance verification.
- 3. Steam generator water level stability and control demonstration.
- 4. Condensate polishing performance testing, when polishing system is available.



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#### LIST OF STARTUP TESTS AND CHECKS

#### TITLE OF TEST OR CHECK

#### TEST OR CHECK OBJECTIVE

- 5. Load rejection testing with condensate polisher, when polishing system is available.
- 6. Steam generator blowdown system flow test at 70% reactor power, when heat recovery portion of the system is available.
- 2. Power Conversion System (Turbine Generator)
  - A. Vibration Frequency and Amplitude

When the main turbine is rolled, vibration readings are monitored, (Turbine vibrations are also monitored throughout the power escalation program). Major equipment (e.g., feedwater pumps and condensate pumps) are operated as they become available and are observed for indications of excessive vibration.

The turbine control system will be demonstrated in turbine operation up to and including a period of operation at synchronous speed. The turbine bypass valves to the condenser and their associated control systems are checked out to verify operability.

The feedwater and condensate pumps are operationally checked out during hot functional testing. During power escalation, the power is increased and the ability of the feedwater pumps and control system to maintain level in the system generators is verified.

The makeup system to the steam generators was checked out during hot functional testing and at power operation. The chemical treatment system is checked out when chemicals are added to the steam generators at heat-up to steaming conditions.



#### B.Turbine Control and Bypass Valves

- C. Feedwater and Feedwater Control System
- D. Makeup Water and Chemical Treatment

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#### APPENDIX D

#### PROCEDURE FOR CONDUCTING THE INTEGRATED STARTUP TEST PROGRAM

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#### PROCEDURE

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#### FOR CONDUCTING THE

#### INTEGRATED STARTUP TEST PROGRAM

#### FOR POST

#### STEAM GENERATOR REPAIR

#### AND

#### PC/M DESIGN CHANGES

TURKEY POINT PLANT FLORIDA POWER AND LIGHT COMPANY MIAMI, FLORIDA

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#### TURKEY POINT UNIT 3 FLORIDA POWER AND LIGHT COMPANY MIAMI, FLORIDA

#### 1.0 PURPOSE AND SCOPE

- 1.1 To provide guidelines for the accomplishment of the tests to be performed prior to plant operations.
- 1.2 To define the phases of the Integrated Startup Test Program.
- 1.3 To delineate the responsibilities of the organizations participating in the Integrated Startup Test Program.
- 1.4 To provide the administrative controls to ensure that the necessary prerequisites are completed prior to commencing the Integrated Startup Test Program.

#### 2.0 <u>REFERENCES</u>

- 2.1 Administrative Site Procedure ASP-11, Construction Turnover
- 2.2 Turkey Point Technical Specifications

#### 3.0 DEFINITIONS

3.1 Integrated Startup Test Program Phases

The Integrated Startup Test Program is divided into two phases. The objectives of each phase are defined below:

- <u>PHASE I</u> Preoperational Tests Those test performed to demonstrate the proper functioning of new, modified and existing equipment, subsystems and systems through and including the Hot Functional Test of the plant prior to fuel loading.
- <u>PHASE II</u> Startup Tests Those tests performed to assure the proper integrated operation of the plant and to demonstrate that the plant can be operated safely as designed from fuel loading to full power operation.

#### 3.2 Plant Procedures

To the extent practical, the Integrated Startup Test Program will be conducted using existing plant procedures as they already include the electrical checks, mechanical checks, calibration checks, hydrostatic tests,



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surveillance requirements, operating parameters, administrative policies, etc., necessary to perform the testing program. Additional procedures will be written as they are required. A copy of the Turkey Point Units 3 & 4 Procedure Index is attached. Three types of these existing plant procedures are defined below.

#### 3.2.1 Administrative Procedure (AP's)

Written instructions which define the method by which the responsible person or persons direct the conduct of plant operations and the means and the limits required for the administration of the plant.

#### 3.2.2 Operating Procedure

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There are three general categories of Operating Procedures:

- 1. Normal Operating Procedures (OP's) which are written instructions which define the normal method, means, and limits of operation, in all modes, of the plant, plant systems, or processes.
- 2. Off Normal Operating Procedure (ONOP's) which specify operator actions for restoring an operating variable to its normal controlled value when it departs from its range, or to restore normal operating conditions following a perturbation. Actions taken under off-normal operating conditions are invoked following an operator observation or alarm to correct a condition which, if not corrected, could degenerate into a condition requiring action under an emergency procedure.
- 3. Emergency Operating Procedures (EOP's) which are written instructions which specify actions, including operator manipulation of controls:
- (1) To avoid further degradation of off-normal conditions which in themselves do not constitute an accident but which could lead to an accident.
  - (2) To reduce the consequences of an accident or hazardous condition which has already occurred.

#### 3.2.3 Maintenance Procedure (MP's)

Written instructions defining the policies and practices by which mechanical, electrical, instrumentation and control and fire protection systems of Units 3 and 4 are kept in a condition of good repair or efficiency so that they may satisfactorily perform their intended functions. These procedures include those activities performed to maintain, modify, or repair nuclear safety related and fire protection equipment. Related activities are those actions taken by operating personnel to determine that a planned maintenance activity can be safely performed under the existing plant conditions. Procedures for these related activities by operations personnel are considered to be operating procedures, but may be included in maintenance procedures.

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#### 3.3 Administative Site Procedures (ASP's)

Written instructions that define the policies and practices of the FPL Construction Department and their associated contractors when performing repairs and design changes on the plant. A copy of the Administrative Site Procedure Index is attached.

#### 3.4 Plant Change/Modifications (PC/M's)

PC/M's are engineered design changes or modifications to plant systems or equipment. A nuclear safety related PC/M is a design change or modification which has a direct effect on those plant features necessary to assure the integrity of the reactor coolant system pressure boundary, the capability to shutdown the reactor and maintain it in a safely shutdown condition, or the capability to prevent or mitigate the consequences of accidents which could adversely affect the environment.

#### 4.0 PARTICIPATING ORGANIZATIONS AND RESPONSIBILITIES

#### 4.1 Plant Nuclear Safety Committee (PNSC)

Consists of management personnel from Turkey Point Plant as designated in the Plant Technical Specifications. The PNSC reviews and recommends approval of all test procedures used during the integrated startup test program, and of items as delineated in the Technical Specifications.

#### 4.2 Turkey Point Operating Staff

Consists of employees engaged in the operation and maintenance of systems, subsystems or portions of the plant under the direction of the Plant Manager-Nuclear.

#### 4.3 <u>Westinghouse Electric Corporation (W)</u>

The term Westinghouse or  $(\underline{W})$  shall be limited to equipment and services furnished as part of or under contract for the nuclear steam supply and turbine generator systems.

#### 4.4 Bechtel Power Corporation (Bechtel)

Bechtel shall furnish Engineering, Construction and Startup services as required by FPL.

#### 4.5 <u>Daniel Construction Company</u> (Daniel)

Daniel shall furnish Construction Quality Control services as required by the FPL Construction QC Department.



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#### 4.6 FPL Construction Department

The Construction Department functions under the direction of the Project Site Manager. The Construction Department is responsible for all work associated with the Steam Generator Repair Project (SGRP) and other design change modifications (PC/M's).

#### 4.7 <u>Quality Control</u> (QC)

#### 4.7.1 FPL Construction QC

The Construction Quality Control Department is responsible for assuring that nuclear safety related components, system or partial systems are released in accordance with existing procedures for testing, that test data documentation is in compliance with contractors' procedure, and that retention control of documentation is adequate. This department is also responsible for verifying that nuclear safety related systems are built in accordance with approved design documents.

#### 4.7.2 FPL Plant QC

The Turkey Point Quality Control Department is responsible for the inspection, and witness of tests required during preoperational and startup testing, and shall maintain control of QC documentation for all activities affecting nuclear power plants. The level and extent of QC involvement is defined in the individual procedures.

#### 4.8 FPL Power Plant Engineering (EPP)

This department is responsible to resolve component and system deficiencies discovered during construction, the Preoperational Test Phase, the Startup Test Phase and during normal plant operations.

#### 4.9 Startup Department

The Startup Department is responsible for the development and implementation of the Integrated Startup Test Program. The department consists of selected plant personnel who are assisted by Bechtel Startup Engineers under the direction of the Startup Superintendent. The Startup Superintendent reports directly to the Plant Manager-Nuclear. The Startup Department is responsible for conducting the tests and checks under their jurisdiction, for assuring that testing is performed in accordance with approved procedures and for reviewing all test data through the Preoperational Test Phase. They will provide technical assistance, as necessary, to the Nuclear Operations Staff during the Startup Test Phase.

#### 4.10 Turnover Committee

The purpose of this Committee is to determine the responsibility for resolving deficiencies discovered after turnover and during the Preoperational Test Phase and the Startup Test Phase.







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#### 5.0 CONSTRUCTION PHASE

The Construction Phase for new, modified or repaired systems is the responsibility of the FPL Construction Department. All activities of the Construction Phase shall be performed in accordance with approved specifications, design change packages, (PC/M's) and procedures, as applicable.

As the construction phase of new, modified or repaired systems is completed, the FPL Construction Department will coordinate walkdown to ensure that the work is completed with the Contractor and FPL Startup Department in preparation for turnover to the FPL Startup Department for Preoperational Testing. Any items that are found to be incomplete or deficient during the walkdowns shall be documented on a Punch List and resolved prior to turnover acceptance by the FPL Startup Department for Preoperational Testing.

The turnover of new, modified or repaired systems shall be accomplished in accordance with an approved Administrative Site Procedure.

#### 6.0 PREOPERATIONAL TEST PHASE

When a system or subsystem is sufficiently complete and related support systems are available so that the intended functions can be performed, the Startup Department will accept the system for preoperational testing. The Preoperational Test Phase is the responsibility of the Startup Department. They will be assisted by the plant staff supplemented by construction crafts personnel as needed. The Startup Department is responsible for documentation of and maintaining a status list of discrepancies determined during preoperational testing. The discrepancies will be documented on a Punch List and the Punch List forwarded to the Turnover Committee for resolution prior to releasing the system for unrestricted operation. The Startup Department is also responsible for assuring that all tests are performed in accordance with approved procedures, reviewing all test data through the preoperational testing phase and writing additional procedures as needed.

Important plateaus in the Preoperational Test Phase are as follows:

- A. Individual Equipment and Systems Tests and Checks
- B. Required Equipment and Systems Aligned and Calibrated
- C. Hot Functional Test of Plant
- D. Primary System Overpressure Test
- E. Cooldown and Release for Startup Test Phase

Upon successful completion of preoperational testing, the Startup Department will review the documentation and test data as required to verify the functional operability of the equipment and systems. Any items found to be incomplete but not limiting for unrestricted operation will be documented on a Punch List and the Punch List forwarded to the Turnover Committee for resolution. The plant is then released to the Turkey Point Operating Staff for the Startup Test Phase.





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#### 7.0 STARTUP TEST PHASE

When the preoperational testing is completed, preparation is made to return the plant to power operation. The systems and equipment tested individually and during the hot functional test are now tested to demonstrate that they can operate safely as an integrated unit from fuel loading to full power operation. The Startup Test Phase is the responsibility of the Turkey Point Operating Staff with technical assistance as required from the Startup Department.

Major milestones in the Startup Test Phase are listed below:

- A. Fuel Load
- B. Leak Rate Testing
- C. Integrated Safeguards Test
- D. Criticality and Low Power Physics Tests
- E. Power Operation

The Startup Test Phase will be performed in accordance with existing plant prodedures that will be changed as necessary to incorporate the operation of new or modified systems and equipment. Additional procedures will be developed should they be required.

Systems that are not used during normal plant operations, but must be in a state of readiness to perform safety functions, are checked under test conditions prior to plant startup. At no time will the safety of the plant be dependent on the performance of untested structures, systems or components.

The Startup Department and the Turkey Point Operating Staff will be responsible to present evidence that the required testing is completed and proper documentation is available.

#### 8.0 ATTACHMENTS

8.1 Turkey Point Units 3 and 4 Procedure Index.

8.2 FPL Construction Administrative Site Procedure Index.

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,			ADMINISTRATIVE
• **	0103.2 -	10/1/81	Responsibilities of Operators on Shift and Maintenance of Operating Logs and Records
	0103.3	5/21/81	Control and Use of Jumpers and Disconnected Leads
	0103.4	10/23/81	In Plant Equipment Clearance Orders
	0103.5	7/30/81	Administrative Control of Valves, Locks and Switches
	0103.6	6/26/80	Reportable Occurrences
	0103.7	6/26/80	Reports Required by Technical Specifications and 10 CFR
	0103.8	8/8/78	Shutdown Rate Guidelines
	0103.9	4/24/79	Facility Staff Qualifications
	0103.10	5/28/81	Using Plant Drawings
	0103.11	8/14/81	Housekeeping
	0103.12	4/2/81	Notification of Significant Event to NRC
	0103.13	5/15/80	Fire Watch Patrol
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	10103.16	11/5/81	Duties and Responsibilities of the Shift Technical Advisor
•	0109.1	3/26/81	Preparation, Revision, and Approval of Procedures
	0109.3 -	11/25/80	On the Spot Changes to Procedures
	0109.6	. 4/30/79	Temporary Procedures
	0110.4	9/3/81	Plant Nuclear Safety Committee - General Procedures
	0149.1	4/23/81	Special Nuclear Material Accountabiliity
	0149.2	.10/31/77	Determination of Average Annual Capacity Factor
	0190.1	8/28/80	Quality Assurance and Quality Control Program and Organization
		• • •	at Turkey Point
	0190.4	.9/24/81	Procurement Document Quality Control
	0190.9	.2/.1/80	Control of Measuring and Test Equipment
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	0190-12	6/18/81	Nonconforming Material, Parts, or Components,
	0190-13	9/24/81	Corrective Action for Conditions Adverse to Quality
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-	0190.16	5/18/79	Scheduling and Surveillance of Periodic Tests and Checks
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	0120-13	2/ 20/ 01	Concretion Systems
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	0100 33	J/ LJ/ /J 5/10/70	Increal Jarcey mazarus: Increation Channes Tests and Experiments
	0130.55	5/10//3	Flortrical Department Instrument Calibration Drogram
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	0150.20		C Equipment
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-	0190.63	9/3/81	Weiging Filler Metal Control Procedures
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	0190.70	8/28/80	Inspection of Maintenance Activities on Nuclear Safety Related
	0190.71 0190.72	<u> 10/23/81 </u> 6/18/81	Qualification of Quality Control Inspectors Receipt Inspection, Identification, and Control of Nuclear Safety Related and Fire Protection Parts, Materials, and Components
Ċ	0190.73 0190.76 0190.77 0190.78	8/28/80 3/5/81 6/18/81 6/20/79	Quality Control Surveillance Program Verification of Steam Generator Tube'Plug Installation Magnetic Particle Inspection (Electromagnetic Yoke Method) Inspection Procedure for Ultrasonic Material Thickness Measurement
C			GENERAL PLANT PROCEDURES
	0202.1 0202.2 0204.1 0204.2	8/20/81 10/15/81 12/18/80 9/3/81	Reactor Startup, Cold Conditions to Hot Shutdown Conditions Unit Startup, Hot Shutdown to Power Operation Secondary Plant Operating Checks and Tests Schedule of Periodic Tests, Checks, Calibrations and Operating
	.0204 <b>.</b> 3 .0204 <b>.</b> 5	5/21/81 ·4/23/81	Initial Criticality After Refueling Nuclear Design Check Tests During Startup Sequence After Refueling
•	0205.1 0205.2 0206.2 0206.3 0206.4	5/2/80 7/30/81 6/18/81 5/21/81	Unit Shutdown, Full Load to Hot Shutdown Conditions Reactor Shutdown, Hot Shutdown to Cold Shutdown Conditions Spent Resin System Refueling Water Storage Tank - Normal Operation Periodic Visual Leak Inspection of Systems Outside the
	0206.5 0208.1 0208.3 0208.4	10/30/80 4/23/81 5/7/81 5/7/81	Containment for Control of Radioactive Material Leakage Visual Leak Inspection of Class III Systems Shutdown Resulting from Reactor Trip or Turbine Trip Annunciator List - Panel A - Reactor Coolant Annunicator List - Panel B - Reactor
	0208.5 0208.6 0208.7 0208.8 0208.9 0208.10	4/30/79 5/7/81 4/10/80 5/7/81 5/7/81 2/12/79	Annunicator List - Panel C - Steam Generator and Reactor Trips Annunciator List - Panel D - Condensate and Feedwater Annunciator List - Panel E - Turbine Generator Annunciator List - Panel F - Electrical Annunciator List - Panel G - Miscellaneous Annunciator List - Panel H - Safety Injection and Auxiliary
Ĺ	0208.11 0208.12 0208.13 0208.14	8/28/80 8/28/80 10/1/81 1/11/80	Annunciator List - Panel I - Station Service Annunciator List - Panel X - Common Annunciator List - Waste/Boron Panels Deviation or Failure of Reactor Protection and Safety Related Hagan Instrumentation Channels
	0209.1 0209.3	9/24/81 9/24/81	Valve Exercising Procedure Inservice Pump Testing Program Implementation Procedure for
	0209.4	11/21/79	Auxillary Feedwater Pumps Inservice Testing - Valve Seat Leakage Testing

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TRAINING PROCEDURES

	0301 0303	. 8/20/81 4/25/77	Licensed Operator Requalification Program Nuclear Operator and Nuclear Turbine Operator Training and
	0004	0/00/01	Retraining Program
	0304	8/20/81	· Plant Iraining
	0305	11/14/80	Emergency leam iraining
	0306	12/12/80	New Employee Indoctrination and Urientation
	0307	6/19/81	Shift Technical Advisor Training Program
			MAINTENANCE PROCEDURE - GENERAL
	0701.1	8/14/81	Welding Job Control List
	0701.2	9/3/81	Quarterly Calibration of Filler Metal (Welding Rod) Ovens
	0703	12/8/78	Maintenance of Copes-Vulcan Air Operated Control Valves
	0707.1	10/4/79	Series G Chempump Removal, Repair, and Replacement
	0707.2	2/24/81	Snubber Inspection and Repair
	0707.8	5/22/80	Periodic Calibration of Instrumentation Used in Station Heat
		-, ==,,	Rate Determination
	0707.9	12/7/79	Inspection of Swagelok Fittings
	0707.10	9/24/81	Reactor Trip and Generator Output Breakers: Inspection and
			Maintenance
	0707.11	3/13/78	Tube Plugging - Auxiliary Heat Exchangers with Stainless Steel
•			Tubes and Flanged Tube Sheets
	0707.13	12/4/79	Fischer-Porter Hagan Pressure Transmitters Alignment and
			Calibration Procedure
	0707.14	11/21/79	Safety and Relief Valve Testing Procedure
	0707.15	5/.19/78	Fischer-Porter Differential Transmitter Alignment,
			Maintenance, and Calibration
	0707.16	1/23/79	Hagan Summators Repairs and Calibration
	0707.17	1/23/79	Hagan Single and Dual Comparators Repairs and Calibration
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	0707.19	5/19 <u>/</u> 78*	We Hagan Manual/Auto Control Station Calibration and Repair
	0707.20	1/23/79	Hagan Isolators Repairs and Calibration
	0707.21	1/23/79	Calibration of Hagan Optimac Analog Computer Elements
	0707.25	7/31/78	Hagan 40V and 45V Loop Power Supply Repair and Calibration
	0707.26	12/15/78	4160 Volt Motor Grounding and Testing Without the Use of
			Manufacturer's Ground and Test Device
	0707.27	12/15/78	4160 Volt Motor Grounding and Testing With the Use of
			Manufacturer's Ground and Test Device
	0707.28	12/15/78	4160/480 Volt Load Center Transformer Grounding and Testing
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	0707.29	9/18/80	Calibration of Rosemount Model 1153 Pressure Transmitter
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	0710.0	4/10/80	Liquid Penetrant Testing
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	0724	4/10/80	Vent and Drain Flange Replacement with Swagelok Plugs
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0726	11/9/78	Velan Pressure Seal Valves - Seal Ring Replacement - Valves 20139, 239, 339
0728	7/22/80	Bushing Replacement on 5KV Penetration Canisters
0729	1/22/81	Safety Related MOV Motor Maintenance
0731	8/31/79	Calibration of Mechanical Department Measuring Test Equipment
0732	12/4/79	QC Check and Replacement of BFD/NBFD Relays in Reactor
0722	10/2/00	Protection and Sareguards Systems
0733	10/2/00	Rayall S MV/1 Amplifier Repairs and Calibration
0734	4/2/01	Safety kelated Supports/ kestraints Removal and Replacement
0735	4/13/81	Emergency Load Sequencer - General Electric Type - HFA Relay
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.1004.4	5/7/81	Overpressure Mitigating System Functional Test of Nitrogen
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1004.5	7/10/81	Reactor Coolant Pressure Isolation Valve Leakage Testing
1005.1	6/11/81	Draining the Reactor Coolant System
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#### REACTOR COOLANT PUMPS

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(	1100.1 - 1107.1	6/6/80 11/15/76	Reactor Coolant Pump Operation Reactor Coolant Pump Seal and Motor Removal
	1107.2	12/4//9	Reactor Loolant Pump Seal Inspection
	1107.5	12/5/80	Reactor Coolant Pump Saal and Motor Deplacement
	1107 7	0/11/20	Peactor Coolant Pump - Defumbiching of Number 2 Seal
	1107.8	7/0/20	Peactor Coolant Pump Motor - Motor Elumber 5 Seat
	1107+0	1/ 5/00	Installation
	1108.1	7/22/80	Reactor Coolant Pump Off-Normal Conditions
		•	PRESSURIZER AND PRESSURIZER RELIEF TANK
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Γ.	1200.1	0/0/00	Pressurizer Steam Space venting
	1207.1	19/24/81	Pressurizer Satety valve, Repair and Setting
	1207.7	4/10/80	Replacement of Pressurizer Mini Spray Valves 524 and 525
-	1207 0	7/31/80	Recallibration of Pressurizer Level Program
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-	1208.2	3/3/76	Pressurizer - Malfunction of Level Control
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<b>—</b>	1407.1	12/7/79	Reactor Vessel - Removal of Missile Shield
(	14072	1/5/79	Removal of Control Rod Drive Mechanism Cooling
<b>~</b> ·	1407.3	8/27/81	Reactor Vessel Removal of Rod Drive Electrical Cables
	1407.4	8/29/78	Reactor Vessel - Removal of Instrument Ports
	1407.5	6/7/78	Reactor Vessel - Removal of Head Insulation
	1407.6	3/26/81 <sup>.</sup>	Installation of Reactor Vessel Cavity Seal Ring
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	1407.8	· 3/26/81	Reactor Vessel - Removal of Vessel Head
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|                  | 11550.34  | 12/4/79            | HP-34 - Matrix Calibration of the Whole-Body Counter           |
|                  | 11550.41  | 10/4/79            | HP-41 - Movement of Material Inside the Radiation Controlled   |
|                  |           | ·······            | Area                                                           |
|                  | 11550.42  | 10/4/79            | HP-42 - Handling and Storage of Radioactive Waste              |
| -                | 11550.43  | 4/25/80            | HP-43 - Inventory and Leak Testing of Sealed Sources           |
|                  | 11550.45  | 12/5/80            | HP-45 - Release of Material from the Radiation Controlled Area |
|                  | 11550.45  | 10/1/81            | nr-40 - Kadioactive waste Snipping                             |
| -                | 11550 4/  | 0/2//01<br>6/20/70 | HP_50 - Use and Maintenance of Protective Clething             |
| (                | 11320+30  | 0/20/13            | newso - use and mathicenance of reductive crouting             |
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|                |           | RADIATION PROTECTION (cont'd)                                                                             |
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| 11550.53       | 4/10/80   | HP-53 - Iodine-131 Air Activity Determination By Use of Whole<br>Body Counter                             |
| 11550.55       | 10/8/81   | HP-55 - Temporary Shielding                                                                               |
| 11550.60       | 7/10/81   | HP-60 - Respiratory Protection Manual                                                                     |
| 11550.61       | 2/17/78   | HP-61 - Full Face Respirator, Air Purifying Type, Scott,<br>Series 801450 and 801500                      |
| 11550.64       | 2/17/78   | HP-64 - Full Face Respirators, Airline Type, Mine Safety<br>Appliances Model 93524/                       |
| 11550.65       | 6/13/80   | HP-65 - Respirator Equipment Maintenance Program                                                          |
| 11550.66       | 2/1/80    | HP-66 - Selection, Use, Issue, Control and MPC Hour<br>Accountability of Respiratory Protection Equipment |
| 11550.67       | 5/29/79   | HP-67 - Full - Face Respirator, Self-Contained Breathing<br>Apparatus Type, Scott Pressur - Pak II        |
| 11550.68       | 10/31/78  | HP-68 - Operation of the Sodium Chloride Respirator Test Booth                                            |
| 11550.70       | 4/25/80   | HP-70 - Decontamination of Personnel                                                                      |
| 11550.71       | 9/7/78    | HP-71 - Decontamination of Tools, Equipment, and Areas                                                    |
| 11550.80       | 9/3/81    | HP-80 - Qualification of Health Physics Personnel                                                         |
| 11550.81       | 10/30/80  | HP-81 - Health Physics Training                                                                           |
| .11550.90      | 6/11/81   | HP-90 - Inventory of Emergency Equipment                                                                  |
| 11550.91       | 10/15/81  | HP-91 - Emergency Radiation Team Response                                                                 |
| 11550.101      | 6/11/81   | HP-101- Radiological Incident Reports                                                                     |
| ,              |           | •                                                                                                         |
| <del>y</del> - |           | SOURCE RANGE NUCLEAR INSTRUMENTATION                                                                      |
| .12104.2       | 10/23/81  | Source Range Nuclear Instrumentation - Periodic Channel<br>Functional Test                                |
| 12107.1        | 11/.13/80 | Source Range Nuclear Instrumentation - High Voltage and Discriminator Voltage Adjustments                 |
| 12107.2        | 12/4/79   | Source Range Nuclear Instrumentation - Adjustments                                                        |
| 12108          | 10/1/76   | Source Range Nuclear Instrumentation Malfunction                                                          |
| •              |           | INTERMEDIATE RANGE NUCLEAR INSTRUMENTATION                                                                |
| . 12204.2      | 10/23/81  | Intermediate Range Nuclear Instrumentation - Periodic Channel                                             |
| 12207.1        | 10/8/81   | Intermediate Range Nuclear Instrumentation - Compensating<br>Voltage Adjustments                          |
| 12207.2        | 11/13/80  | Intermediate Range Nuclear Instrumentation - Checkout and<br>Adjustments                                  |
| 12208          | 3/3/76    | Intermediate Range Malfunction                                                                            |

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|          |           | POWER RANGE NUCLEAR INSTRUMENTATION                                                                                                      |
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| 12304.2  | 10/23/81  | Power Range Nuclear Instrumentation - Periodic Channel                                                                                   |
|          | - <u></u> | Functional Test                                                                                                                          |
| 12304.3  | 6/18/81   | Power Range Nuclear Instrumentation - Shift Checks and Daily<br>Calibrations                                                             |
| 12304.4  | 6/3/75    | Power Range Nuclear Instrumentation - Channel Check and Calibration                                                                      |
| 12304.6  | 6/18/81   | Power Range Nuclear Instrumentation - Calculation of Target                                                                              |
| 12304.8  | 7/31/80   | Inducing Xenon Oscillation to Produce Various Incore Axial                                                                               |
| 12207 2  | 1/22/01   | UTTSELS<br>Deven Bange Overseven This Vist Dance Adjustments                                                                             |
| 12207.2  | 4/23/81   | Power Range Overpower Irip High Range Adjustments                                                                                        |
| 12307.5  | 11/13/80  | Instrumentation, Axial Flux Deviation Process Instrumentation<br>to OPSP and OTSP, and Nuclear Power Range Axial Flux Deviation<br>Alarm |
| 12308    | 10/1/81   | Power Range Nuclear Instrumentation Malfunction                                                                                          |
| 12308.2  | 8/6/81    | Power Range Nuclear Instrumentation - Verification of Upper,<br>Lower, and Channel Deviation Alarms                                      |
| 12401.2  | 12/15/77  | Incore Nuclear Instrumentation - Determination of Upper and<br>Lower Limit Switch Settings                                               |
| .12404.1 | 8/28/80   | Power Distribution Surveillance Using Incore Movable Detector                                                                            |
| 12404.2  | 1/22/81   | Determination of Effective Avial Deaking Factor                                                                                          |
| 12404.3  | 3/26/81   | Implementation of Augmented Surveillance                                                                                                 |
| 12407.1  | 3/26/81   | Patracting and Incerting Incore Instrumentation Thimbles                                                                                 |
| 12407 2  | 19/7/70   | Income Flux Detector Drive Machanism Densin and Detector                                                                                 |
| 16407.06 | 12/1/15   | Replacement                                                                                                                              |
| 12407.3  | 9/19/78   | Incore Flux Mapping System Thimble Cleaning and Lubrication                                                                              |
|          |           | INCORE THERMOCOUPLES                                                                                                                     |
| 12607.2  | 2/15/80   | Incore Thermocouple - Maintenance Checkout                                                                                               |
|          |           | EXCORE NUCLEAR INSTRUMENTATION                                                                                                           |
| 12702 1  | 0/10/70   | Normal Alignment of Deactivity Computer                                                                                                  |
| 12707 1  | 5/22/80   | 'Excore Nuclear Instrumentation Descript Dre Installation and                                                                            |
| 12/0/•1  | 5/22/00   | Doct Installation Inspections and Test of Detectors                                                                                      |
| 12707.2  | 4/6/79    | Excore Nuclear Instrumentation Installation of Detectors                                                                                 |
|          |           | CONTAINMENT LEAKAGE SURVEILLANCE                                                                                                         |
| 13100.11 | 3/16/70   | Containment Integrated Loak Pate Test Unit 2                                                                                             |
| 13100.2  | 1/12/21   | Containment Integrated Leak Nate Test - Unit 3                                                                                           |
| 13104.1  | 12/15/78  | Containment Durne Valve Leak Nate Jest - Unit 4                                                                                          |
| 13108.1  | 7/10/81   | loss of Containment Integrity                                                                                                            |
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| NUMBER                                                           | DATE                                                              |                                                                                                                                                                                                                                                                                                                                                          |
|                                                                  | • •                                                               | CONTAINMENT TENDONS                                                                                                                                                                                                                                                                                                                                      |
|                                                                  |                                                                   | PENETRATIONS                                                                                                                                                                                                                                                                                                                                             |
| 13404.1<br>13404.2                                               | 10/30/80<br>12/15/78                                              | Containment Boundary Isolation Valves - Local Leak Rate Test<br>Electrical Penetration Canisters - Local Leak Rate Test                                                                                                                                                                                                                                  |
|                                                                  |                                                                   | CONTAINMENT HATCHES                                                                                                                                                                                                                                                                                                                                      |
| 13513<br>13514.1<br>13514.2<br>13523<br>13531.1<br>13537.2       | 11/21/79<br>8/20/81<br>6/25/81<br>2/15/80<br>1/23/79<br>3/19/75   | Personnel Access Hatch - Operating Instructions<br>Personnel and Emergency Air Locks - Local Leak Rate Tests<br>Containment Access Hatch - Local Leak Rate Test (Vacuum Test)<br>Emergency Access Hatch - Operating Instruction<br>Equipment Access Hatch - Local Leak Rate Test<br>Personnel and Emergency Access Hatches - Preventative<br>Maintenance |
|                                                                  |                                                                   | PROCESS INSTRUMENTATION                                                                                                                                                                                                                                                                                                                                  |
| .14004.1<br>14004.2<br>14004.3<br>14004.4                        | 6/13/80<br>3/26/81<br>9/18/81<br> 11/5/81                         | Steam Generator Protection Channels - Periodic Test<br>Reactor Coolant Flow Protection Channels - Periodic Test<br>Tavg and Delta T Protection Channels - Periodic Test<br>Pressurizer Pressure and Water Level Protection Channels -<br>Periodic Test                                                                                                   |
| 14004.5<br>14007.5<br>14007.6<br>14007.7<br>14007.11<br>14007.12 | 4/3/80<br>10/23/81<br>.6/18/81<br>10/23/81<br>4/23/81<br>.5/18/81 | Seismograph - Periodic Test<br>Recalibration of High Delta T Alarm Setpoint<br>Tref Program - Recalibration<br>Tavg Alarms - Recalibration<br>Reactor Coolant Temperature<br>Reactor Coolant Flow Instrumentation Calibration During                                                                                                                     |
| 14007.13                                                         | <u> 11/12/81 </u><br>4/2/81                                       | Refueling<br>Pressurizer Water Level Instrumentation Calibration<br>Pressurizer Pressure Instrumentation Calibration During<br>Refueling                                                                                                                                                                                                                 |
| 14007.15                                                         | 12/5/80<br>8/27/81                                                | Rod Position Indication Instrumentation Calibration During<br>Refueling<br>Steam Generator Level Feedwater Flow and Steam Flow                                                                                                                                                                                                                           |
| 14007.17<br>14007.18<br>14007.19                                 | 12/7/79<br>9/18/80<br>11/14/77                                    | Instrumentation Calibration During Refueling<br>Charging Flow Instrumentation Calibration During Refueling<br>Residual Heat Removal Pump Flow Instrumentation Calibration<br>Boric Acid Tank Level Instrumentation Calibration During<br>Perfueling                                                                                                      |
| 14007.20<br>14007.21                                             | 12/22/78<br>· 7/31/78                                             | Refueling Water Storage Tank Level Instrumentation Calibration<br>Volume Control Tank Level Instrumentation Calibration During                                                                                                                                                                                                                           |
| 14007.22                                                         | 12/22/78                                                          | Containment Pressure (Wide and Narrow Range) Instrumentation<br>Calibration During Refueling                                                                                                                                                                                                                                                             |
| 14007.23                                                         | 11/14/77                                                          | Containment Sump Level Instrumentation Calibration During                                                                                                                                                                                                                                                                                                |

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|         |                                  | *                                        |                                                                                                                                                                |                                               |
|         |                                  |                                          | PROCESS INSTRUMENTATION (cont'd)                                                                                                                               |                                               |
| •       | 14007.24<br>14007.25             | 11/14/77<br>9/18/80                      | Accumulator Level Instrumentation Calibra<br>Accumulator Pressure Instrumentation Calibra                                                                      | ation During Refueling<br>ibration During     |
|         | 14007.26                         | 8/27/81                                  | Steam Line, Steam Header and Turbine 1st                                                                                                                       | Stage Pressure                                |
|         | 14007.27                         | 1/22/80                                  | Boric Acid Blender Control System Instru<br>Refueling                                                                                                          | mentation Test During                         |
|         | 14007.28<br>14007.29<br>14007.30 | <u> 11/12/81 </u><br>1/30/79<br>11/12/81 | Tavg and AT Control Loops Calibration Ch<br>Recalibration of Over Power Delta T Setp<br>Auxiliary Feedwater Flow Indication and<br>Instrumentation Calibration | eck and Adjustments<br>oint (OPSP)<br>Control |
|         | 14007.31                         | 11/12/81                                 | Replacement of NBFD Relays in Reactor Pre<br>Safeguards System During <u>Refueling</u>                                                                         | otection and                                  |
| ÷       | •                                |                                          | WATER TREATMENT PLANT                                                                                                                                          |                                               |
|         | 15100 ·                          | 8/27/81                                  | Water Treatment Plant (WTP) - Pretreatmen<br>Shutdown                                                                                                          | nt Section Startup and                        |
|         | 15103                            | 8/27/81                                  | WTP Pretreatment Section - Normal Operat                                                                                                                       | ion                                           |
| -       | 15200<br>.15209                  | · 6/25/81<br>7/1/81                      | WIP Demineralizer Section - Normal Operation<br>WTP Demineralizer Section - Abnormal Oper<br>Supply to Carbon Filters                                          | ration - Raw Water                            |
| Ţ       | 15307.1<br>15307.2               | 1/23/79<br>.10/15/81.                    | Primary Water Makeup Pump Motor Overhaul<br>Replacing Carbon in Activated Carbon Pres                                                                          | and Maintenance<br>ssure Filters              |
| Ļ.      | •                                | •                                        | PRIMARY WATER SYSTEM                                                                                                                                           |                                               |
|         |                                  |                                          | FIRE PROTECTION SYSTEM                                                                                                                                         |                                               |
|         | 15500<br>15524                   | .3/13/81<br>6/18/81                      | Fire Protection Program<br>Fire Protection Pump and Power Supply -                                                                                             | Periodic Test                                 |
|         | .15525                           | 8/20/81<br>.10/24/78                     | Fire Protection Underground Piping - Per<br>Fire Pump Motor Overhaul                                                                                           | idic Flow Test                                |
|         | 15537.1                          | 10/1/81                                  | Fire and Smoke Detection System - Semi-A                                                                                                                       | nnual `Test                                   |
|         | 15537.3                          | 5/2/80                                   | Surveillance of Penetration Fire Barrier                                                                                                                       | s (Fire Stops)                                |
|         | 115537.4                         | 11/5/81                                  | Annual Fire Hose Hydro Test                                                                                                                                    | ing Instructions                              |
| (       | 15543                            | 10/8/81                                  | Normal Operation of Main, Auxiliary, Sta<br>H <sup>2</sup> Seal Oil System Deluge Fire Protection                                                              | rtup Transformers and<br>n System             |
|         | 15549.1                          | 3/25/77                                  | Resetting Deluge Fire Protection System .<br>SUPROTEX Automatic Valve                                                                                          | After Actuation of                            |
|         |                                  |                                          | INSTRUMENT AIR SYSTEM                                                                                                                                          |                                               |
|         | 15600.1                          | 6/18/81                                  | Instrument Air System - Operating Instru                                                                                                                       | ctions                                        |
|         | 15608.1<br>15650                 | 2/24/81<br>_9/3/81                       | Loss of Instrument Air<br>Breathing Air System Operating Instruction                                                                                           | on                                            |
| Ċ       |                                  |                                          | POLAR CRANE                                                                                                                                                    | -                                             |
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|           |                     | FUEL HANDLING                                                                            | -                   |
| •         |                     |                                                                                          |                     |
| 16000.1   | 4/24/79             | Limits and Precautions for Handling Fuel As                                              | semblies            |
| 16001.2   | 8/20/81             | Technical Specification Surveillance Requir<br>Refueling                                 | ements for Core     |
| 16002.5   | 3/13/81             | Refueling Core Shuffle                                                                   |                     |
| 16002.6   | 6/11/81             | Preparations and Precautions for Refueling                                               | Fuel Shuffle        |
| 16002.7   | 10/30/80            | Refueling Pre-Shuffle in Spent Fuel Pit                                                  |                     |
| 16004.1   | 12/15/78            | Fuel Transfer Tube Flange Local Leak Rate T                                              | est                 |
| 16004.2   | 6/11/81             | Refueling System Interlocks Verification                                                 |                     |
| 16007.1   | 12/20/79            | Fuel Transfer Tube - Installation of Blind                                               | Flange ·            |
| 16008.1   | 10/23/75            | Accident Involving New Fuel                                                              |                     |
| 16008.2   | 10/23/75            | Accident Involving Spent Fuel                                                            |                     |
| 16009.1   | 12/4/79             | Receipt and Handling of New Fuel Containers                                              |                     |
| 16009.2   | 2/15/80             | Site Removal of New Fuel Assemblies from Sh                                              | ipping Containers   |
| 2000502   | 2, 20, 00           | and Handling of Shipping Containers                                                      | ipping concernere   |
| 16009.6   | 11/5/81             | On-Site Unpacking Inspection and Manual Loa                                              | ding of Burnable    |
| 1000510   |                     | Poison Rod Assemblies                                                                    |                     |
| 16100     | 12/5/80             | Fuel Transfer System Normal Operation                                                    |                     |
| 16104 1   | 12/18/80            | Fuel Transfer System - Periodic Test                                                     | - ,                 |
| 16122     | 2/26/01             | Filling the Defueling Canal and Deactor Def                                              | ueling Cavity       |
| 16125 1   | 5/20/01             | Draining Defueling Cavity Using DHD System                                               | dering cavity       |
| 16125 2   | 5/14/01<br>11///77  | Draining Refueling Cavity below the Desctor                                              | Voccol Elando       |
| 10123.2   | 11/4///             | Lovel with the Departon Coelant Durin Tank                                               | vesser ridige       |
| 16195 3   | 711/1/70            | Design the Pofueling Cavity and SED Caral                                                | holow the Paseton   |
| 1012203   | 11/.1//9            | Vessel Flance Level to the PWST using the F                                              | mergency SFD Dumn   |
| 16120 1   | 10/12/70            | Durifying Deactor Coolant and Defueling Cau                                              | ity During          |
| 10169+1   | 10/.12/79           | Pafualing Using a CVCS Mixed Red Deminerali                                              | 70r                 |
| 16120 3   | 10/1//76            | Peactor Cavity Filtration System - Operation                                             | n and Filton        |
| 10123+3   | 10/ 14/ /0          | Chappeout                                                                                | n and i ticei       |
| 16200     | 12/1/70             | Manipulaton Crane Operating Instructions                                                 |                     |
| 16200 1   | 2/20/21             | Manipulator Grane and DCC Change Eixture De                                              | riodic Tests        |
| 16207 1   | 12/7/70             | Manipulaton Grane Domoval and Installatio                                                | n of Innon Mast an  |
| 10207.1   |                     | Grinner                                                                                  | in or timer mast an |
| 16300     | 5/21/76             | Spont Fuel Dit Bridge Crane Operating Instr                                              | uctions             |
| 16300 1   | 11/12/20            | Spent fuel Fit bildge Glane Operating Instr<br>Spent Fuel Dit Bridge Chang Dariedic Tect |                     |
| 16400     | 0_10/13/70          | New Eugl Flowston Operating Instructions                                                 |                     |
| 16400     | 12///70             | New Fuel Elevator Operating Instructions                                                 |                     |
| 16404+1   | 1/20/70             | New Fuel Elevalui - Feriouri lest                                                        |                     |
| 16464+2   | 1/.30//9<br>5/21/76 | New Fuel Monorall Hoist - Periouic lest                                                  | 0.05                |
| 16500     | 5/21/70<br>0///77   | New Fuel Phonorall Hoist Operating Instruction                                           |                     |
| 16500 1   | 12/4/70             | New Fuel Bridge Grane Operating Instruction                                              | 2                   |
| 16700     | 12/4//9             | Cask Change Change Instructions                                                          |                     |
| 16701 1   | 5/11/3<br>6/11/01   | Cantry Crane - Increation and Dreventative                                               | Maintonanco         |
| 16702 1   | 0/27/01             | Soont Eugl Chipmont Using Model NEC & Chipm                                              | na muchanue         |
| 16702 2   | 0/2//01             | Spent Fuel Transfor                                                                      | Thy bask            |
| 16702 2   | 4/16///<br>7/16/01  | Sport Fuel Transfer (NAC and NEC Chinging C                                              | ack)                |
| 16702+3   | 10/01               | Thimble Dlug Transfer                                                                    | ashj                |
| 16702 5   | 10/25/70            | , Spant Fuel Shipmont Haing National Load Ind                                            | ustrios (NET)       |
|           | 10/73//9            | - JUGHL FUEL JULUNGHL USINU NALIUNAI LEAQ (NO                                            | USLEIES LIVEL       |

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# FUEL HANDLING (cont'd)

| 16707.1<br>16708.1<br>16711.1<br>16900.1<br>16900.3<br>16900.4<br>16900.5                                                                                                                          | 8/11/78<br>6/23/76<br>10/9/74<br>3/26/81<br>1/30/75<br>9/17/74<br>7/31/78                                                                                                                                            | Operation of Spent Fuel Building Sliding Door<br>Spent Fuel Cask Emergency Cooling<br>Irradiation Surveillance Capsule Handling Cask DOT-55-14300<br>Uncoupling Full Length Control Rods<br>Thimble Plug Assembly Handling Tool<br>Spent Fuel Assembly Handling Tool - Operating Instructions<br>Irradiation Specimen Handling Tool Operation and Specimen                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                          |
|----------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------|----------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------|-----------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------|
| 16900.6<br>16900.9                                                                                                                                                                                 | 9/19/74<br>3/3/76                                                                                                                                                                                                    | Container Transfer<br>Guide Tube Cover Handling Tool - Operating Instructions<br>RCC Change Fixture Operation                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                       |
| 16900.11<br>16900.12<br>16900.13<br>16900.15<br>16900.16                                                                                                                                           | L .12/5/80<br>9/17/74<br>5/14/81<br>1/20/78<br>9/3/81                                                                                                                                                                | Burnable Poison Rod Assembly Handling Tool Manual Type<br>New Fuel Assembly Handling Fixture Operating Instructions<br>Core Mapping Following Core Loading<br>Coupling of Full Length Control Rods<br>Rod Cluster Control Change Tool                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                               |
| •                                                                                                                                                                                                  |                                                                                                                                                                                                                      | SAMPLE SYSTEM                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                       |
| 18000                                                                                                                                                                                              |                                                                                                                                                                                                                      | DIGITAL DATA PROCESSING SYSTEM                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                      |
|                                                                                                                                                                                                    | `.                                                                                                                                                                                                                   | EMERGENCY PROCEDURES                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                |
| 20000<br>20001<br>20002<br>20003<br>20004<br>20005<br>20101<br>20102<br>20103<br>20104<br>20105<br>20106<br>20107<br>20109<br>20109<br>20110<br>20111<br>20112<br>20113<br>20125<br>20126<br>20201 | 3/5/81<br>3/5/81<br>3/5/81<br>3/5/81<br>6/11/81<br>4/25/80<br>4/23/81<br>8/14/81<br>4/23/81<br>10/8/81<br>10/8/81<br>3/26/81<br>3/26/81<br>3/26/81<br>3/26/81<br>3/26/81<br>3/26/81<br>3/26/81<br>3/26/81<br>3/26/81 | <pre>(E-0) - Immediate Actions and Diagnostics<br/>(E-1) - Loss of Reactor Coolant<br/>(E-2) - Loss of Secondary Coolant<br/>(E-3) - Steam Generator Tube Rupture<br/>Loss of Offsite Power<br/>Control Room Inaccessability<br/>Loss of Feedwater Flow or Steam Generator Level<br/>Duties of Emergency Coordinator<br/>Duties of an Individual Who Discovers an Emergency Condition<br/>Classification of Emergencies and Criteria for Evacuation<br/>Emergency Roster<br/>On-Site Support Centers<br/>Natural Emergencies<br/>Fire and Explosion Emergencies<br/>Criteria For, and Conduct of Local Evacuation<br/>Criteria For, and Conduct of Owner Controlled Area Evacuation<br/>Re-entry<br/>Communications Network<br/>Maintaining Emergency Preparedness, Emergency Exercises,<br/>Drills, Tests and Evaluations<br/>On-Site Emergency Organization<br/>Loss of Coolant Accident Dose Calculation<br/>Maintaining Emergency Preparedness - Radiological Emergency<br/>Plan Training</pre> |
| 20201                                                                                                                                                                                              | 3/20/81.                                                                                                                                                                                                             | Plan Training Emergency Preparedness - Radiological Emergency                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                                       |



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| • 6        | PROCEDURE<br>NUMBER | DATE     | Page 22<br>TITLE 11/20/81                                                         |
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| C          |                     | ,        | OFF-SITE EMERGENCY ORGANIZATION                                                   |
|            | 1101                | 9/8/81   | Duties of the Emergency Control Officer - Off-Site Emergency Organization         |
|            | 1102                | 9/8/81   | Duties of the Recovery Manager, Off-Site Emergency<br>Organization                |
| <u> </u>   | 1103                | 4/1/81   | Duties of the Emergency Information Manager; Off-Site<br>Emergency Organization   |
|            | 1104                | 4/1/81   | Duties of the Emergency Security Manager                                          |
|            | 1105                | 4/1/81   | Duties of the Emergency Technical Manager                                         |
|            | 1105                | 4/1/81   | Duties of the Governmental Affairs Manager                                        |
|            | 1107                | 4/1/81   | Duties and Responsibilities of the Emergency Plan<br>Administrator                |
| <b>-</b> . | 1211 -              | -4/1/81  | Activation and Use of the Emergency News Center (Turkey Point)                    |
|            | 1212                | 9/8/81   | Activation and Use of the Interim Emergency Operations<br>Facility (Turkey Point) |
|            | .1301               | 10/23/81 | Emergency Roster - Off-Site Emergency Organization                                |

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| <i>"</i> ^ | PROCEDURE<br>NUMBER | DATE    | TITLE                                                                                                           | Page 23<br>10/2/81                    |
|------------|---------------------|---------|-----------------------------------------------------------------------------------------------------------------|---------------------------------------|
|            | ,                   | •       |                                                                                                                 | L                                     |
| )          |                     |         | WELDING PROCEDURES (Limited Distribution O                                                                      | nly)                                  |
|            | WP-1A               | 8/14/81 | Manual Shielded Metal-Arc Process - For Wel<br>Plate and Piping                                                 | ding Carbon Steel                     |
|            | WP1-AT              | 8/14/81 | Manual, Combination Process - Welding Tungs<br>Shielded Metal Arc-Materials Group P1 to P1                      | ten Inert Gas and                     |
|            | WP6-A               | 8/14/81 | Manual Shielded Metal-Arc Process - For Wel<br>Steel Plate and Piping                                           | ding 12% Chromium                     |
|            | WP1-8-AT(1)         | 8/14/81 | Manual, Combination Process Welding Tungste                                                                     | n Inert Gas and<br>1                  |
| ,          | WP-8-AT             | 8/14/81 | Manual, Combination Process Welding, Tungst<br>Shielded Metal Arc Materials Group P8 to P8                      | en Inert Gas and                      |
|            | WP1-T               | 8/14/81 | Gas Tungsten - Arc Welding - For Materials                                                                      | in the P-1 Grouping                   |
|            | WP1-8-T             | 8/14/81 | Gas Tungsten - Arc Welding - For Materials                                                                      | in the P-1 and P-8                    |
| ł <b>=</b> | WP8-T               | 8/14/81 | Gas Tungsten - Arc Welding - GTAW Process F<br>Welded in the P-8 Grouping of Section IX of<br>and Pressure Code | or Materials to be<br>the ASME Boiler |

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October 6, 1981

## PLANT CONSTRUCTION

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## ADMINISTRATIVE SITE PROCEDURES INDEX

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| C          | ASP-16 | Calibration and Control of Measuring and Test Equip.                   | 1                  |
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