

## Summary Report of Open Generic Communication Projects (As Of 11/30/2017)

### Division of Inspection and Regional Support

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*(For additional information contact Erika Lee at: erika.lee@nrc.gov)*

CAC/EPIDS NUMBER	TITLE AND SUMMARY	AGE (Mon)	CAC/EPIDS OPENED
MF3949	<p><b>RIS 'RIS 2005-29 Rev 1 Anticipated Transients That Could Develop into More Serious Events'</b></p> <p><i>This RIS update is intended to clarify the guidance contained in previously issued RIS 2005-29. RIS 2005-29 was initially issued to discuss concerns with demonstrating that certain events would not escalate into more serious events.</i></p>	44.0	04/6/2014
MF4346	<p><b>RIS 'Pressure Boundary Leakage'</b></p> <p><i>This RIS was initially intended to discuss technical specification definitions related to reactor coolant system pressure boundary leakage but was placed on hold pending industry development of Technical Specification Task Force (TSTF) Traveler 554, "Revise Reactor Coolant Leakage Requirements." The NRC was recently informed that industry no longer plans to submit TSTF Traveler 554 for review, therefore NRC is again evaluating whether to develop this RIS.</i></p>	42.0	06/30/2014
MF5661	<p><b>RIS 'Safety-Related SSCs in Service Beyond Service Life'</b></p> <p><i>This RIS is intended to reiterate existing requirements related to dispositioning information pertaining to the capability of safety-related structures, systems, and components (SSC) to perform their safety functions. Also addressed are instances where a licensee becomes aware of information pertaining to the time period that a safety-related SSC is installed that may impact its ability to perform its safety function(s).</i></p>	34.0	02/03/2015
MF7230	<p><b>RIS 'Identifying and Reporting Human Factor Incidents Under 10 CFR 50.73'</b></p> <p><i>This RIS is intended to remind licensees of the requirements regarding reporting human performance incidents under 10 CFR 50.73, "Licensee event report system," and how to properly report those matters. This RIS specifically focuses on 10 CFR 50.73 (b)(2)(ii)(J), which requires a narrative description for each human performance related root cause discussing the causes and circumstances.</i></p>	23.0	01/13/2016
MF8453	<p><b>RIS 'Functionality of Non-Tech Spec Support Systems as They Relate to Operability of Tech Spec Systems'</b></p> <p><i>This RIS is intended to clarify the applicability of the single failure criterion as it relates to technical specification systems, structures and components with support systems not covered in technical specifications</i></p>	13.0	10/11/2016
MF9358	<p><b>RIS 'Human Reliability and Human Performance Database, Rev. 1'</b></p> <p><i>This revision of RIS 2017-01 will supersede the information contained in existing RIS 2017-01. It is intended to inform licensees about the NRC's Scenario, Authoring, Characterization, and Debriefing Application software for operator simulator training and might solicit licensees, on a voluntary basis, to use the software.</i></p>	9.0	03/06/2017
MF9405	<p><b>RIS 'NRC Staff Position on Applying Surveillance Requirements 3.0.2 and 3.0.3 to Administrative Controls Program Tests'</b></p> <p><i>This RIS will supersede the guidance provided in RIS 2012-10.</i></p>	8.0	03/22/2017

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MF9464	<p><b>RIS ‘Use of EPRI/NEI Joint Task Force Report, “Guideline on Licensing Digital Upgrades: EPRI TR-102348, Revision 1, NEI 101-01: A Revision of EPRI TR-102348 to Reflect Changes to the 10 CFR 50.59 Rule, Rev. 1’</b></p> <p><i>This supplement to RIS 2002-22 is intended to clarify the NRC’s endorsement of NEI 01-01 by providing additional guidance for the qualitative assessment used to provide reasonable assurance that a digital modification will exhibit a low likelihood of failure.</i></p>	8.0	03/27/2017
MF9558	<p><b>RIS ‘Clarification of Visual Examination Requirements for Reactor Upper Heads to Detect Signs of Leakage</b></p> <p><i>This RIS is intended to clarify the requirements for bare metal visual examination.</i></p>	8.0	04/05/2017
MF9830	<p><b>RIS ‘Common Violations of the 10 CFR Part 37 Requirements and Related Guidance Documents’</b></p> <p><i>This RIS is being developed to inform addressees of common violations that the NRC has identified during inspections conducted to verify compliance with the requirements of 10 CFR Part 37 in order to raise awareness of these particular violations and reduce their occurrence, and to provide an overview of the difference between Part 37 and the series of security orders issued prior to the promulgation of Part 37.</i></p>	5.0	06/12/2017
MF9974	<p><b>BL ‘Degradation of Anchor Darling Double Disc Gate Valve Stem to Disc Connection</b></p> <p><i>This Bulletin is intended to request information regarding Anchor/Darling double disc gate valves in light of recent operating experience that involved degradation of these valves.</i></p>	4.0	07/20/2017
MG0020	<p><b>IN ‘Boiling-Water Reactor Dryout Indications Occurring During Normal Operation’</b></p> <p><i>This IN is intended to inform addressees of recent operating experience related to numerous dryout indications which occurred over multiple cycles at the Leibstadt Nuclear Power Plant (NPP), a BWR/6 in Switzerland.</i></p>	4.0	07/28/2017
MG0051	<p><b>IN ‘Noble Fission Gas Release During Spent Fuel Cask Loading Operations’</b></p> <p><i>This IN is intended to inform the addressees of operating experiences related to noble fission gas releases during spent fuel loading operations.</i></p>	4.0	08/04/2017
MG0146	<p><b>RIS ‘Electronic Signature Use for Medical Licensee Internal Documents Required by NRC Regulation’</b></p> <p><i>This RIS is intended to supply information to assist medical use licensees in complying with the current NRC requirements related to the use of electronic signatures in internal licensee electronic documents and to clarify what constitutes a signature on electronic documents that medical use licensees are required to sign and maintain.</i></p>	3.0	08/24/2017
MG0173	<p><b>IN ‘Failure of Operators to Trip Plant When Experiencing Unstable Conditions’</b></p> <p><i>This IN is intended to inform addressees of several reactor events where operators failed to place the plant in a stable, shutdown condition in a timely manner despite indications that the reactor was not in a stable condition.</i></p>	3.0	08/29/2017

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MG0234	<p><b>RIS ‘Process for Scheduling and Allocating Resources in Fiscal Year 2020 for the Review of New Licensing Applications for Light-Water Reactors and Non-Light-Water Reactors’</b></p> <p><i>This RIS is being developed to request voluntary information that will assist the NRC in determining resource and budget needs as well as aligning the proper allocation and utilization of resources to support applicant submittals, future construction-related activities, and other anticipated light water reactor and non-light water reactor licensing and design certification rulemaking actions for Fiscal Year 2020.</i></p>	2.0	09/13/2017
MG0257	<p><b>IN ‘Operating Experience Regarding Failure to Meet Technical Specifications Requirements’</b></p> <p><i>This IN informs addressees of several reactor events where operators failed to ensure that requirements of the plant technical specifications were met as plant conditions changed.</i></p>	2.0	09/21/2017
L2017-CRS-0062	<p><b>IN ‘Supplier Oversight Issues Identified During NRC Vendor Inspections’</b></p> <p><i>This IN is intended to make addressees aware of recent experiences related to inadequate oversight of suppliers that provide basic components to NRC-licensed facilities.</i></p>	2.0	10/02/2017
L2017-CRS-0004	<p><b>RIS ‘Reporting Requirements for Components Exported Under General License Provisions’</b></p> <p><i>This RIS is intended to clarify the reporting requirements for nuclear facilities and equipment as required by 10 CFR 110.54 and the International Atomic Energy Agency for the Application of Safeguards in the United States.</i></p>	2.0	10/03/2017
L2017-CRS-0058	<p><b>IN ‘Testing and Operation-Induced Degradation of 3-Stage Target Rock Safety Relief Valves’</b></p> <p><i>This IN is intended to make addressees aware of recent operating experiences related to Model 0867F 3-stage Target Rock safety relief valves (SRV). Operating experience has shown that limited flow testing of these valves can result in damage to internal valve components.</i></p>	2.0	10/09/2017
L2017-CRS-064	<p><b>IN ‘Process Assumptions Resulting in Unanalyzed and Improperly Analyzed Conditions Related to Long-Term Fissionable material Accumulation in Fuel Cycle Facilities’</b></p> <p><i>This IN is intended to inform addressees of potential safety concerns that have been contributors to several recent safety-significant events regarding analyzed conditions related to long-term fissionable material accumulation in fuel cycle facilities.</i></p>	0.5	11/15/2017

**Closed Generic Communication Projects Since Last Report  
Division of Inspection and Regional Support**

<b>CAC/EPIDS NUMBER</b>	<b>TITLE AND SUMMARY</b>	<b>Date Opened</b>	<b>Date Closed</b>
<b>MF7597</b>	<b>RIS 'Clarifying Appropriate Methods for Resolving Errors in AP1000 Certified Design Information'</b>  <i>This RIS was intended to document information about AP1000 design errors that met the criteria set forth in Interim Staff Guidance DC/COL ISG-011, "Finalizing Licensing-Basis Information." Management decided that the issuance of a RIS was not the most efficient means of addressing this issue. The staff expects to engage industry on this topic during an upcoming public meeting.</i>	04/20/2016	11/22/2017