CATEGORY 1

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ACCESSION NBR:9804280038 DOC.DATE: 98/04/23 NOTARIZED: NO DOCKET #
FACIL:50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
AUTH.NAME AUTHOR AFFILIATION
KINGSEED,J. Indiana Michigan Power Co.
SAMPSON,J.R. Indiana Michigan Power Co.
RECIP.NAME RECIPIENT AFFILIATION

SUBJECT: LER 98-020-00:on 980408,interim LER - determined that containment recirculation sump pH upper limit exceeded. Caused by analysis input omission.Condition remains under evaluation to determine appropriate action.W/980423 ltr.

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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106

AEP
INDIANA
MICHIGAN
POWER

April 23, 1998

United States Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

98-020-00

Sincerely,

J. R. Sampson
Site Vice President

/mbd

Attachment

c: A. B. Beach, Region III
E. E. Fitzpatrick
P. A. Barrett
S. J. Brewer
R. Whale
D. Hahn
Records Center, INPO
NRC Resident Inspector

1/1
IE 22

NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	Page 1 of 1
Donald C. Cook Nuclear Plant - Unit 1	50-315	

TITLE (4)
Interim LER -Containment Recirculation Sump pH Upper Limit Exceeded Due to Analysis Input Omission

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	08	98	98	-- 020 --	00	04	23	98	Cook - Unit 2	50-316
									FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	5
POWER LEVEL (10)	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)

20.2201(b)	20.2203(a)(3)(i)	50.73(a)(2)(iii)	73.71(b)
20.2203(a)(1)	20.2203(a)(3)(ii)	50.73(a)(2)(iv)	73.71c
20.2203(a)(2)(i)	20.2203(a)(4)	50.73(a)(2)(v)	OTHER
20.2203(a)(2)(ii)	50.36(c)(1)	50.73(a)(2)(vii)	(Specify in Abstract below and in Text, NRC Form 366A)
20.2203(a)(2)(iii)	50.36(c)(2)	50.73(a)(2)(viii)(A)	
20.2203(a)(2)(iv)	50.73(a)(2)(i)	50.73(a)(2)(viii)(B)	
20.2203(a)(2)(v)	X 50.73(a)(2)(ii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
Mr. Jeb Kingseed, Nuclear Safety Analysis Manager	616 697-5106

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X YES (If yes, complete EXPECTED SUBMISSION DATE).	NO		07	31	98

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 9, 1998, an ENS notification was made in accordance with 10 CFR 50.72(b)(2)(i), for a condition which was found while both units were shutdown, which had it been found while the reactor was in operation, could have resulted in the nuclear plant being in an unanalyzed condition. This notification is a result of an issue discovered during a safety system functional inspection of the containment spray system.

It has been determined that the Westinghouse method for calculating the maximum pH in the containment sump after an accident might result in a pH that is too basic. While responding to an AEP audit question, Westinghouse discovered that the sodium hydroxide contained within the ice condenser ice beds had not been included in any of the BORDER analysis performed for Cook Plant. BORDER is a computer program used to determine the pH in the containment sump. The impact of this is that the actual containment sump pH value following a loss of coolant accident exceeds the maximum pH limit in the Bases for Technical Specification (T/S) 9.5. Preliminary calculations indicate a pH value of approximately 9.7 is possible. The T/S is based on Branch Technical Position MTEB 6-1 on the pH for emergency cooling water for PWRs. The reference states that the pH should be in the high end of the range of 7.0 to 9.5 for greater assurance that stress corrosion cracking will not occur. The reference does not give any guidance on what the affects would be if 9.5 is exceeded, thus the unanalyzed condition. The significance of the impact of the pH above the range in the T/S requires further evaluation.

This condition remains under evaluation to determine appropriate action to rectify this issue. Additionally, this issue is being evaluated for potential 10CFR Part 21 reportability. An update to this interim LER will be submitted by July 31, 1998.