April 26, 1993

Docket Nos. 50-315 and 50-316

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See Meeting Summory

LICENSEE: Indiana Michigan Power Company (IMPC)

FACILITY: D. C. Cook, Units 1 and 2

SUBJECT: MEETING SUMMARY OF APRIL 7, 1993

On April 7, 1993, representatives of the American Energy Power Services Corporation (parent company of IMPC) and Foxboro met with members of the NRC staff and contractors from Idaho National Engineering Laboratory (INEL) in Rockville, Maryland to discuss the planned installation of digital processing equipment in the reactor protection systems (RPS) for both units. A list of attendees is enclosed (Enclosure 1).

This meeting was a followup meeting from ones that were held in April and December 1992 and was timed to coincide with the inclusion of the INEL contractors to the project for review and audit purposes. The licensee provided a brief history of the project, and discussed the proposed schedule and project organization. A Foxboro representative described the instrumentation that will be installed and the licensee then briefly discussed its plans for conducting the verification and validation process for the project. The final part of the licensee's presentation was associated with how the pertinent documentation is arranged and the scope of their license amendment package. The licensee described the type and volume of information that was available to support the staff's review process. This included an extensive effort to cross-reference the information to any staff issues or questions that have arisen in the past. A copy of the licensee's presentation is enclosed (Enclosure 2).

The INEL contractors asked a number of questions of a technical nature to become better familiarized with the status of the licensee's project. The meeting then concluded, though several of the licensee's technical representatives remained to more thoroughly answer some of the questions posed to them by the staff and INEL contractors. Original signed by

William M. Dean, Sr. Project Manager Project Directorate III-1 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Enclosures: As stated

cc w/enclosures: See next page

Office	LA:PDIII-1	PM: PDDII-1	D:PDIII-1
Name	MShuttleworth	WDean/vsb;sw	LMarsh _
Date	-1 <u>2/-/92</u>	9/16/45	4 70/47 12/ /92

7/22/93 9305030321 930426 PDR ADDCK 05000315

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Docket File w/encls. NRC & Local PDRs PDIII-1 RDG File <u>w/o enclosure 2</u> T. Murley F. Miraglia J. Partlow J. Roe J. Zwolinski L. Marsh W. Dean OGC E. Jordan, MNBB 3701 J. Wermiel 10/D/24 J. Mauck 8/H/3 S. Rhow 8/H/3 ACRS (10) G. Grant, EDO W. Shafer, RIII E. Schweibinz, RIII B. Jorgensen, RIII

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Indiana Michigan Power Company

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Regional Administrator, Region III U.S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, Illinois 60137

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Attorney General Department of Attorney General 525 West Ottawa Street Lansing, Michigan 48913

Township Supervisor Lake Township Hall post Office Box 818 Bridgman, Michigan 49106

Al Blind, Plant Manager Donald C. Cook Nuclear Plant Post Office Box 458 Bridgman, Michigan 49106

U.S. Nuclear Regulatory Commission Resident Inspector Office 7700 Red Arrow Highway Stevensville, Michigan 49127

Gerald Charnoff, Esquire Shaw, Pittman, Potts and Trowbridge 2300 N Street, N. W. Washington DC 20037

Mayor, City of Bridgman Post Office Box 366 Bridgman, Michigan 49106

Special Assistant to the Governor Room 1 - State Capitol Lansing, Michigan 48909

Nuclear Facilities and Environmental Monitoring Section Office Division of Radiological Health Department of Public Health 3423 N. Logan Street P. O. Box 30195 Lansing, Michigan 48909

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Donald C. Cook Nuclear Plant

Mr. S. Brewer American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

E. E. Fitzpatrick Indiana Michigan Power Company c/o American Electric Power Service Corporation 1 Riverside Plaza Columbus, Ohio 43216

ENCLOSURE 1

#### <u>April 7, 1993</u>

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#### D. C. Cook, Units 1 and 2 RPS Modification Meeting

#### List of Attendees

Name

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File + wear

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#### <u>Organization</u>

Bill Dean Ed Schweibinz Joe Joyce Sang Rhow Paul Loeser Jim Stewart Jeb Kingseed Bill Sotos Doug Malin Hal Hoffman Robert Carruth Mark Wilken James Keiper Jeff Hansen Bob Richards Senior Project Manager, PDIII-1/NRC Senior Project Engineer, RIII/NRC HICB/NRR HICB/NRR HICB/NRR AEPSC AEPSC AEPSC AEPSC AEPSC AEPSC Foxboro EG&G/INEL EG&G/INEL



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USNRC/INEL/AEPNO

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APRIL 7, 1993 MEETING

# **REACTOR PROTECTION SYSTEM**

# **PROCESS INSTRUMENTATION REPLACEMENT**

WHITE FLINT OFFICE

ROCKVILLE, MD

# PROJECT HISTORY, SCOPE AND STATUS...... W. G. SOTOS

## TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION......W.G. SOTOS

OVERVIEW OF FOXBORO INSTRUMENTATION ......J. T. KEIPER

SOFTWARE VERIFICATION AND VALIDATION ........ H. H. HOFFMAN

OVERVIEW OF SUBMITTAL ...... M. A. WILKEN



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# PROJECT HISTORY, SCOPE AND STATUS

W.G. SOTOS



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**INITIAL STUDY (INITIATED 1988)** 

EQUIPMENT ·

H-LINE EQUIPMENT END OF LIFE

**DECLINING VENDOR SUPPORT** 

DECLINING ELECTRONIC PIECE/PART LEVEL SUPPORT

**DESIGN PHILOSOPHY** 

**USE MATURE PRODUCT LINES** 

**NO FUNCTION CHANGES** 

NO CHANGES IN SEPARATION

**NO CHANGE IN DIVERSITY** 

NO DIGITAL COMMUNICATION NETWORKS

**USE EXISTING FIELD WIRING** 

MINIMIZE IMPACT ON TECHNICIANS AND OPERATORS

MINIMIZE IMPACT ON PLANT PROCEDURES

MAXIMIZE PRE-INSTALLATION TESTING







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SCOPE

REACTOR PROTECTION SETS

CONTROL GROUP PROCESS INSTRUMENTATION

CONTROL PANEL HARDWARE

WATER SYSTEMS INSTRUMENTATION

FABRICATION START (NOVEMBER 1990)

FACTORY ACCEPTANCE TEST START (SEPTEMBER 1991)

C E REACTOR PROTECTION SET INVOLVEMENT ENDED (MARCH 1992)

NO INVOLVEMENT ON SAFETY RELATED WORK

TAYLOR EQUIPMENT IS KEPT IN NON-SAFETY WORK

FOXBORO CONTRACT (APRIL 1992)

SCOPE

**REACTOR PROTECTION SETS** 

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CURRENT PROJECT

FOXBORO

**REACTOR PROTECTION SYSTEM** 

TAYLOR

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CONTROL GROUP PROCESS INSTRUMENTATION

WATER SYSTEMS INSTRUMENTATION

CONTROL PANEL HARDWARE

AMSAC (CHANGED FROM FOXBORO TO MAINTAIN DIVERSITY)

#### SCOPE OF WORK

REPLACE EXISTING ANALOG FOXBORO H-LINE EQUIPMENT WITH NEW FOXBORO SPEC 200 AND SPEC 200 MICRO HARDWARE

IMPACTS 13 REACTOR PROTECTION PROCESS INSTRUMENTATION CABINETS EACH UNIT





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# TECHNICAL AREAS OF INTEREST

- ENVIRONMENTAL QUALIFICATION
- SEISMIC QUALIFICATION
- EMI/RFI SUSCEPTIBILITY
- POWER
- INDEPENDENCE/DIVERSITY
- FAILURE MODES AND EFFECTS
- RELIABILITY
- FUNCTIONALITY
- VERIFICATION AND VALIDATION
- TESTING

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**PROJECT ORGANIZATION AND MANAGEMENT** 

AEPNO DEDICATED PROJECT TEAM

MANAGEMENT

LEAD ENGINEER

**SUPPORTING I&C ENGINEERS** 

**CONTRACTING CONSULTING ENGINEERS** 

MECHANICAL AND ELECTRICAL DESIGNERS

PLANT PROJECT ENGINEER

PLANT I&C PERSONNEL

**QUALITY ASSURANCE** 

NUCLEAR SAFETY AND LICENSING

**CURRENT STAFFING** 

**17 FULL TIME PERSONNEL** 

28 PART TIME (GREATER THAN 25% OF AVAILABLE TIME)

FORMAL STATUS AND ACTION ITEM LISTS REGULARLY

PROJECT SCHEDULE (UNIT 2 UNLESS NOTED)

**RPS DOCUMENTATION** 

VENDOR DRAWING/DESIGN

FACTORY ACCEPTANCE TEST

COMPLETION

AEP DRAWING/DESIGN COMPLETION

PRE-INSTALLATION TEST

COMPLETION

PLANT PROCEDURE

COMPLETION

**UNIT 1 INSTALLATION** 

COMPLETE

COMPLETE

MAY 1993

**JULY 1993** 

**AUGUST 1993** 

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**DECEMBER 1993** 

**FEBRUARY 1994** 

# TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION

W.G. SOTOS

# TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE (C.Y.) APPLICATION

# FUNCTIONAL

# HARDWARE/HARDWARE APPLICATIONS

# SOFTWARE/SOFTWARE APPLICATIONS



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## TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION

#### **FUNCTIONAL - REACTOR PROTECTION AND ESF**

**CONNECTICUT YANKEE** 

PRESSURIZER LEVEL

PRESSURIZER PRESSURE

**REACTOR COOLANT FLOW** 

S/G NARROW RANGE LEVEL

S/G STEAM FLOW

<u>D.C. COOK</u>

PRESSURIZER LEVEL

PRESSURIZER PRESSURE

**REACTOR COOLANT FLOW** 

S/G NARROW RANGE LEVEL

S/G STEAM FLOW





# **TECHNICAL DESCRIPTION AND COMPARISON WITH**

### CONNECTICUT YANKEE APPLICATION

(CONTINUED)

#### **FUNCTIONAL - REACTOR PROTECTION AND ESF**

**CONNECTICUT YANKEE** 

COOK NUCLEAR PLANT

S/G PRESSURE

S/G PRESSURE

FEEDWATER FLOW

FEEDWATER FLOW

T<sub>AVG</sub>, ΔT (THOT, TCOLD)

**CONTAINMENT PRESSURE** 

TURBINE FIRST STAGE PRESSURE T<sub>AVG</sub>, ΔT (THOT, TCOLD)

CONTAINMENT PRESSURE

TURBINE IMPULSE PRESSURE



TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION

(CONTINUED)

# FUNCTIONAL - OTHER SAFETY RELATED APPLICATIONS

CONNECTICUT YANKEE WIDE RANGE RCS TEMPERATURE WIDE RANGE RCS PRESSURE COOK NUCLEAR PLANT WIDE RANGE RCS TEMPERATURE WIDE RANGE RCS PRESSURE RWST LEVEL CONTAINMENT LEVEL AUXILIARY FEEDWATER FLOW TSAT INPUT SELECT CONDENSATE STORAGE TANK LEVEL

# TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION (CONTINUED)

#### FUNCTIONAL DIFFERENCES

# **REACTOR PROTECTION AND ESF - NONE**

OTHER SAFETY RELATED - ADDITIONAL APPLICATIONS FOR REG GUIDE 1.97



## **TECHNICAL DESCRIPTION AND COMPARISON WITH**

### **CONNECTICUT YANKEE APPLICATION**

# (CONTINUED)

HARDWARE

CONNECTICUT YANKEE

**ANALOG INPUT MODULES:** 

N-2AI-12V

N-2AI-P2V N-2AI-T2V N-2AI-C2L

**ANALOG OUTPUT MODULES:** 

N-2AO-VAI N-2AO-V2I N-2AO-V2H

**CONTACT OUTPUT MODULES:** 

N-2AO-L2C-R

D.C. COOK

**ANALOG INPUT MODULES:** 

N-2AI-H2V N-2AI-P2V N-2AI-T2V N-2AI-C2L

**ANALOG OUTPUT MODULES:** 

N-2AO-V2H (CUSTOM) CONTACT OUTPUT MODULES:

N-2AO-L2C-R



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TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION (CONTINUED)

HARDWARE

**CONNECTICUT YANKEE** 

D.C. COOK

DIGITAL MODULES:

N-2CCA-S N-2CCA-D N-2CCA-S

**DIGITAL MODULES:** 

N-2CCA-D

**POWER SUPPLIES/DISTRIBUTION:** 

N-2AX+DP11

N-2ARPS05-A6 N-ARPS-A6 **POWER SUPPLIES/DISTRIBUTION:** 

N-2AX+DP11 N-2AX+DP10-E N-2ARPS05-A6

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## TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION (CONTINUED)

HARDWARE

CONNECTICUT YANKEE

<u>D.C. COOK</u>

**NESTS:** 

N-2ANU-DM

**NESTS:** 

**RACKS:** 

N-2ANU-DM

**RACKS:** 

**NEW N-2ES SERIES** 

**EXISTING AMCO FX78 SERIES** 



#### CONNECTICUT YANKEE APPLICATION

(CONTINUED)

HARDWARE DIFFERENCES

ANALOG INPUT MODULES - 10 - 50 - MA VS 4 - 20 MA

ANALOG OUTPUT MODULES - N - 2AO - V2H MODIFIED TO PROVIDE ISOLATION

**CONTACT OUTPUT MODULES - NONE** 

**DIGITAL MODULES - NONE** 

**POWER SUPPLIES/ DISTRIBUTION** 

MINOR FUSING DIFFERENCES IN POWER DISTRIBUTION MODULES USED 75 VDC POWER SUPPLY REQUIRED

**NESTS - NONE** 

**RACKS - EXISTING VS NEW** 



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## TECHNICAL DESCRIPTION AND COMPARISON WITH CONNECTICUT YANKEE APPLICATION (CONTINUED)

SOFTWARE

#### **CONNECTICUT YANKEE**

SOFTWARE BLOCKS USED:

CALC

ALRM

LLAG

GATE

<u>D.C. COOK</u>

SOFTWARE BLOCKS USED:

CALC

ALRM

LLAG

GATE

RAMP

SSEL

• VERIFICATION AND VALIDATION PER ANSI/IEEE-ANS-7-4.3.2



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#### **OVERVIEW OF FOXBORO INSTRUMENTATION**

J. T. KEIPER

INDUSTRY CONSULTANT

THE FOXBORO COMPANY







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**SPEC 200 NEST ASSEMBLY** 

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# **TYPICAL MODULE INSTALLATION**



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TYPICAL ANALOG I/O MODULE

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CONTACT OUT MODULE

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# SPEC 200 MICRO MODULES



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# SPEC 200 MICRO



# SPEC 200 MICRO CONTROL CYCLE

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#### **CONFIGURABLE BLOCK TYPES**

CONTROL:

PID	Ξ	<b>PROPORTIONAL/INTEGRAL/DERIVATIVE</b>
		CONTROLLER
TUNE	=	EXACT-TUNING EXTENDER (PID)
NONL	=	NON-LINEAR EXTENDER (PID)
INT		INTEGRAL ONLY CONTROLLER
AMB	Ξ	AUTOMATIC/MANUAL STATION WITH BIAS
RTIO	=	RATIO INPUT AND CONVERSION

INPUT AND CONVERSION:

MIB	Ξ	MULTIPLE INPUT BLOCK
CHAR	=	PIECEWISE LINEAR CHARACTERIZER

DIGITAL/LOGIC:

DIN	=	DIGITAL INPUT
DOUT	=	DIGITAL OUTPUT
GATE	Ħ	MULTIPLE GATE
SEQ	Ξ	SEQUENCER

#### **DYNAMIC COMPENSATION:**

LLAG = LEAD/LAG DYNAMIC COMPENSATOR DTIM = DEAD TIME

#### **MISCELLANEOUS:**

SWCH	=	SWITCH
SSEL	=	SIGNAL SELECTOR
ALRM	=	ALARM AND LIMITER
RAMP	=	UNIVERSAL RAMP GENERATOR
ΓIMR	=	TIMER
ACUM	=	ACCUMULATOR
CALC	=	CALCULATOR



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Power Distribution via SPEC 200 Nest Power Distribution Module and Multi-Nest Power Supply

### SPEC 200/SPEC 200 MICRO SIMPLIFIED BLOCK DIAGRAM

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High Pressurizer Pressure SPEC 200/SPEC 200 MICRO



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H-Line





PQ-455

Over Power Delta T Over Temperature Delta T SPEC 200/SPEC 200 MICRO



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	1977	78	79	80	້ 81	82	83	84	85	86	87	88	89	90	91	92	Total
N-2AI	695	411	482	690	563	725	253	592	* 416	364	324	268	961	120	214	297	7375
N-2AO	219	340	377	797	658	774	306	686	379	518	512	412	971	· 218	337	701	8205
N-2AX	1708	1080	1570	2195	2041	2649	1044	2169	1121	1341	676	846	2907	903	2871	497	25618
N-2CCA										73	151	75	390	59	68	221	1037
N-2CDS				•						10	53	20	115	54	4	13	269
N-2AN	195	- 263	281	416	291	262	86	214	176	188	146	80	507	58	113	185	3461
N-2ES	້ 59	56	22	40	31	17	7	88	13	6	7	3	31	1	2	6	389
N-2AR	· 46	59	75	75	110	90	36	51	38	40	49	22	107	41	6	57	902
N-2AC	47	137	125	140	127	282	54	91	134	47	19	13	59	18	27	4	1324
N-2AP	535	450	643	767	. 596	588	269	516	207	177	170	131	472	98	127	97	5843
N-227						51	20	87	80	107	52	14	94	21	37	11	574
N-E27						16	9	20	59	15	41	9	. 3	9	33	2	216
N-25X	43	40	74	105	230	101	185	94	213	147	87	46	567	294	1244	14	3484

#### PRODUCT EXPERIENCE

#### N-SPEC 200

> 100 NUCLEAR PLANTS

> 50,000 NUCLEAR MODULES

#### N-SPEC 200 MICRO

**30 NUCLEAR PLANTS** 

> 1000 NUCLEAR MODULES

SPEC 200 MICRO

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> 400 USERS

> 14,700 MODULES



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### SOFTWARE VERIFICATION AND VALIDATION

H. H. HOFFMAN



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#### VERIFICATION

A PROCESS THAT GOES ON DURING THE ENTIRE LIFE CYCLE. THE PROCESS IS THAT OF DETERMINING WHETHER OR NOT THE PRODUCT OF A GIVEN PHASE OF THE SOFTWARE DEVELOPMENT CYCLE FULFILLS THE REQUIREMENTS IMPOSED BY THE PREVIOUS PHASE.

#### VALIDATION

THE TEST AND EVALUATION OF COMPLETED SOFTWARE TO ENSURE CORRECTNESS AND COMPLIANCE WITH SOFTWARE REQUIREMENTS AND APPLICABLE STANDARDS, RULES, AND CONVENTIONS. FOR EXAMPLE, SOFTWARE MAY BE VALIDATED EITHER: (1) MANUALLY (A NON-PROGRAMMABLE CALCULATOR IS CONSIDERED MANUAL), (2) BY BENCHMARKING AGAINST ANOTHER CONTROLLED AND VALIDATED SOFTWARE PROGRAM, (3) BY COMPARING THE SOFTWARE OUTPUT TO EXPERIMENTAL DATA, OR (4) BY BENCHMARKING AGAINST A TEST CASE WITH KNOWN RESULTS.
## HARDWARE QUALIFICATION

IEEE 603-1980 STANDARD CRITERIA FOR SAFETY SYSTEMS FOR NUCLEAR POWER GENERATING STATIONS.

IEEE 344-1975 RECOMMENDED PRACTICES FOR SEISMIC QUALIFICATION OF CLASS 1E EQUIPMENT FOR NUCLEAR POWER GENERATING STATIONS.

ANSI/IEEE ANS-7-4.3.2-1982, APPLICATION CRITERIA FOR PROGRAMMABLE DIGITAL COMPUTER SYSTEMS IN SAFETY SYSTEMS OF NUCLEAR POWER GENERATING STATIONS.



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## SOFTWARE

## **VERIFICATION & VALIDATION**

## PLAN



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# IEEE -730.1 - 1989 PLAN USED FOR SOFTWARE DEVELOPMENT AND MAINTENANCE

SECTION	3.3	-	MANAGEMENT
	3.4	-	DOCUMENTATION
	3.4.1	-	GOVERNING DOCUMENTATION
	3.4.2	-	MINIMUM DOCUMENTATION
			REQUIREMENTS
	3.5	-	STANDARDS PRACTICES AND
			CONVENTIONS
	3.8	-	PROBLEM REPORTING AND
			CORRECTIVE ACTION
	3.9	-	TOOLS TECHNIQUES AND
			METHODOLOGIES
	3.10	-	CODE CONTROL
	3.11	-	MEDIA CONTROL
	3.12	-	SUPPLIER CONTROL
	3.13	-	RECORDS COLLECTION
			MAINTENANCE AND RETENTION

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## SOFTWARE

## **VERIFICATION & VALIDATION**

## REPORT

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#### COMPLIANCE TO

ANSI/IEEE - ANS - 7 - 4.3.2

COMPUTER SYSTEM REQUIREMENTS HARDWARE REQUIREMENTS - INPUT/OUTPUT - DESIGN FEATURES - INITIALIZATION REQUIREMENTS - SECURITY AND DIAGNOSTICS - HUMAN-FACTORS ENGINEERING - INTERRUPT FEATURES

#### COMPLIANCE TO

ANSI/IEEE - ANS - 7 - 4.3.2 Continued

#### • SOFTWARE REQUIREMENTS

- PROCESS INPUTS

- OPERATING SYSTEM, UTILITY ROUTINES, OTHER AUXILIARY PROGRAMS

- ALGORITHMS

- DATA FILES

- OUTPUTS

- INITIALIZATION

- PROGRAM LOGIC FOR FAILURES

- OPERATOR INTERFACE

- IN-SERVICE TEST FEATURES

- TIMING REQUIREMENTS

- PROCESSING IDLE TIME

- SECURITY REQUIREMENTS

- CONFIGURATOR

#### **COMPLIANCE TO**

ANSI/IEEE - ANS - 7 - 4.3.2 Continued

- HARDWARE/SOFTWARE INTEGRATION
- SOFTWARE DEVELOPMENT
- HARDWARE/SOFTWARE INTEGRATION
- COMPUTER SYSTEM VALIDATION
- VERIFICATION/VALIDATION
- COOK NUCLEAR PLANT SPECIFIC APPLICATION

## **OVERVIEW OF SUBMITTAL**

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M. A. WILKEN

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OBJECTIVE

# • DESCRIBE HOW NRC CONCERNS HAVE BEEN ADDRESSED

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#### **CE/TAYLOR EQUIPMENT AUDITS**

DURING THE AUDIT OF THE PROPOSED CE/TAYLOR EQUIPMENT THE NRC REQUESTED EXTENSIVE AMOUNTS OF DOCUMENTATION AND ASKED US TO RESPOND TO NUMEROUS QUESTIONS.

REQUESTS WERE WRITTEN IN OUR AUDIT NOTES AND IN INFORMAL INFORMATION EXCHANGES BETWEEN AEPNO AND THE NRC.

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#### NRC MEETINGS

- APRIL 1992
  - NRC REQUESTED US TO ADDRESS THE ISSUES
     RAISED DURING THEIR REVIEW OF THE
     CE/TAYLOR EQUIPMENT.
  - NRC IDENTIFIED OTHER AREAS OF INTEREST, E.G., FUNCTIONAL DIVERSITY
  - INFORMATION TO BE DOCKETED.
- DECEMBER 1992
  - PROVIDED STATUS ON THE RPS PROJECT TO THE NCR SINCE THE APRIL 1992 MEETING
  - DETERMINE THE FORMAT OF THE LICENSING SUBMITTAL DATED DECEMBER 15, 1992

# ACTIONS TAKEN TO RESPOND TO NRC INFORMATION REQUESTS

- COMPILED OUR NOTES AND LISTS OF QUESTIONS.
- DELETED QUESTIONS NOT RELEVANT TO THE FOXBORO EQUIPMENT, I.E., THOSE SPECIFIC TO THE CE/TAYLOR EQUIPMENT.
- CREATED A FILING SYSTEM TO IDENTIFY THE SPECIFIC ISSUES THAT NEEDED TO BE ADDRESSED.
- PROVIDED SPECIFIC RESPONSES TO THE APPLICABLE QUESTIONS (APPROXIMATELY 150) IN OUR ENGINEERING DOCUMENTATION.
- PREPARED A "ROAD MAP" TO FACILITATE NRC REVIEW OF OUR ENGINEERING DOCUMENTATION.



#### As of <u>12/15/92</u>

#### **DOCUMENTATION TRACKING LIST**

The following items are those that the NRC expects to be available for review (after November 30, 1992). These items were identified during the April 21, 1992 meeting at NRR. The items, primary responsible group, and due date are as follows:

- 1. Background Information
  - A. Functional Drawings
    - 1. Existing (AEPSC I&C CMPLT) 2-98501 thru 2-98516
    - 2. New (Foxboro CMPLT) See Appendix G \*

3200 Functional Requirement Summary (AEPSC) & CMPLT) Report No. 2985-DPS-01

- B. System Architecture Description
  - 1. Existing
    - a. Instruction Manuals (AEPSC 1&C CMPLT) Foxboro Manuals Volumes I thru IV
    - b. Simple Block Diagram (AEPSC I&C CMPLT) No Document No.
  - 2. New
    - a. List of Applicable Instruction Manuals (Foxboro CMPLT) No Document no. - Attached as Appendix A
    - b. Applicable Instruction Manuals (AEPSC I&C CMPLT) See Appendix A
    - c. Write General System Description (Foxboro CMPLT) 0310-4120

page 1 of 22 12/ 15/ 92



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	FILE NO.		
1. BACKGROUND INFORMATION			
A. FUNCTIONAL DRAWINGS			
1. For any "functional" or "logic" changes, provide diagrams and an explanation.	1A2 - Functional Drawings 1B2d-General System Description		
2. Provide the loop diagrams, and functional diagrams for the RPS channels including the current revision of the modification package.	Existing: 1-D-1, 1-A-1 New: 1A2, 1D2		
3. Provide rack internal and external arrangement drawings.	16		
4. Provide plant one-line power drawings (120v) feeding cabinets (new and existing).	Existing: 1E1 New: 1E2		
7. Provide the existing elementary drawings - RPS only.	1-D-1 , .		
8. Provide the new elementary drawings - RPS only.	1-D-2		
9. Provide rack assembly drawings.	1-G		
B. System Architectural Descriptions			
1. Detailed drawings of the Foxboro Spec 200/Spec 200 Hicro Electronics.	1-B-2b, drawings contained in Instruction Hanuals		
2. Provide Foxboro RPS equipment information including the Factory Acceptance Tests, Service Bulletins, trouble reports/maintenance requests.	Available at Foxboro		
3. Provide assurance that the system, when installed, will meet the requirements of the plant design basis, including the FSAR and licensing commitments.			
- Origin of functional requirements	1-B1-B existing 1-B2-E new		
- Provide functional block diagrams before/after the upgrade	V&V report in file 24A5, figures 1 and 2, and Section 2.2.		
- Demonstrate equivalency, are there differences in the functional requirements.	1. 50.59 review 2. 4/21/92 presentation		
4. Provide setpoints, gains, constants, resets, etc., traceable to design basis.	24.6.2		

Page 1 of 17 12/15/92

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**DOCUMENTATION TRACKING LIST (CONT)** 

- G. Approved U2 Set 1 Foxboro Drawings/ Complete Set (AEPSC I&C CMPLT) \* See Appendix F & G \*
- H. Approved U2 Set 1 Sys Decrip/ Config Data Base (AEPSC I&C CMPLT) \* Foxboro Doc No. TP-150 \* Foxboro Doc No. DB-151 \*
- 2. Reg Guides and Standards applicable
  - A. Provide summary in revised RFC packet (AEPSC I&C See Item 36.C) RFC 2985 Rev 1 packet \*
  - B. Explain how we comply with each and any differences that apply.
    - 1. Explain existing variances (AEPSC I&C CMPLT) Report No. 2985-NCF-01 \*
    - 2. Explain any new equipment variances (Foxboro None Identified/No Specific Action)
  - C. Describe any special conditions to our license and impact on standards compliance (AEPSC NS CMPLT) See RFC DC-12-2985 packet Safety Review Memo file 36:C:

page 3 of 22 12/15/92

- 3. Provide a specific comparison to Haddam Neck for both hardware and software
  - A. Update information provided at April 21, 1992 NRC meeting (AEPSC I&C CMPLT) No document number
    - B. Provide applicable MI's as references (see Item 1.B.2.b)

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	FILE HO.	
- Outputs; isolation, separation and barriers	1. V&V report in file 24A5, Sect. 2 and Sect. 3.1.3, discussion on IEEE-603-1980.	
	2. 281, pages 8 and 9 of 13	
<ul> <li>Independence of safety/non-safety signals; isolation separation and barriers</li> </ul>	1. V&V report in file 24A5, Sect. 2 and Sect. 3.1.3, discussion on IEEE-603-1980.	
-	2. 281, pages 8 and 9 of 13	
C. Details on how we meet all of the Reg. Guide, IEEE Standards, etc., referenced in our purchase order specification.	24A5 and 2B1	

ſ	3.	Specific Comparison	to Haddam Neck for Bo	th Hardware and Softw	are	3-A	 
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4. Temperature and Humidity			
A. Provide the evaluation that established the design temperature limits to be experienced within the enclosure for the new instrumentation.	4B page 2 of 4 Specifies equipment requirements		
	The overall environmental report will not be written until February 1993.		
B. Provide thermal test reports and plans.	4A 4B-reviews 4A 4C-add'l plan 4D-appv'l for 4C plan Items 4C and 4D are test procedures which will		
×	be performed to demonstrate that the heat rise in the circuit cards is acceptable.		
C. Describe any anomalies that may have occurred during all of the Eaxbard testing and how they were resolved.	4B		
	An additional heat rise test will be done in January 1993.		

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#### CONCLUSIONS

• NRC STATED CONCERNS HAVE BEEN ADDRESSED IN ENGINEERING DOCUMENTATION

LICENSE AMENDMENT PER 10 CFR 50.59 (b) (3) (c) HAS
 BEEN SUBMITTED FOR DOCKETING WITH
 ENGINEERING DOCUMENTATION.

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