

## NOTICE OF VIOLATION

Indiana Michigan Power Company  
Donald C. Cook Nuclear Power Plant

Docket No. 50-315; 50-316  
License No. DPR-58; DPR-74

During an NRC inspection conducted from May 5 - 23, 1997, violations of NRC requirements were identified. In accordance with the "General Statement of Policy and Procedure for NRC Enforcement Actions," NUREG-1600, the violations are listed below:

1. 10 CFR 50, Appendix B, Criterion V, "Instructions, Procedures, and Drawings," requires, in part, that activities affecting quality be prescribed by procedures of a type appropriate to the circumstances and be accomplished in accordance with these procedures.

Contrary to the above,

- A. On May 10, 1997, the inspectors identified that safety related Temporary Modification (TM) 1-95-1, which did not require an outage for restoration, had been assigned a (administrative) date of August 27, 1996, but had not been made a permanent installation through a design change or been removed as of May 10, 1997 as required by Plant Managers Procedure (PMP) 5040.MOD.OO1, "Temporary Modifications," Revision 7.
- B. On May 7, 1997, the inspectors identified that an activity affecting quality, the deenergization of the DG2AB inverter, was completed without placing 2-DGAB-INV-CB2 and 2-DGAB-INV-CB1 to off, contrary to steps 2.1.2 and 2.1.3 of procedure 02-OHP 4021.032.008 "Aligning DG2AB Subsystems For Standby Operation," Revision 4, Attachment 7.

This is a Severity Level IV violation (Supplement I).

2. 10 CFR 50, Appendix B, Criterion III, requires, in part, that measures be established to assure that the design basis are correctly translated into specifications, drawings, procedures, and instructions. Design control measures shall provide for verifying or checking the adequacy of design.

Contrary to the above, design control measures were not adequate to assure that the design basis was correctly translated into design modification documents:

- a. On May 7, 1997, the inspectors identified that calculation DC-D-1-SI-F101, "Stress Analysis & Load Generation for System 1-SI-F101 Per 12-MM-590," used the wrong moment arm and had a missing reaction force and moment.
- b. On May 13, 1997, the inspectors identified that incorrect and non-conservative design input was used for the motor-driven auxiliary feedwater pump start time in the Safety Review Memorandum for the Setpoint Values for the Time Delay Pickup Relays in the AFWS Flow Retention Circuits, dated January 15, 1997, for design change package 12-DCP-0817, "Revise Aux. Feedwater Flow Retention Circuit."



- c. On May 13, 1997, the inspectors identified that a calculation for the seismic design adequacy of minor modification 12-MM-337 was not performed. The design package for 12-MM-337 indicated this calculation existed as DC-D-12-ES-116.

This is a Severity Level IV violation (Supplement I).

The inspection showed that adequate corrective actions had been taken for both examples in the first violation. Consequently, no response is required to this violation. For violation No. 2, pursuant to the provisions of 10 CFR 2.201, Indiana Michigan Power Company is hereby required to submit a written statement or explanation to the U.S. Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, D.C. 20555 with a copy to the Regional Administrator, Region III, and a copy to the NRC Resident Inspector, at the facility that is the subject of this Notice, within 30 days of the date of the letter transmitting this Notice of Violation (Notice). This reply should be clearly marked as a "Reply to a Notice of Violation" and should include for each violation: (1) the reason for the violation, or, if contested, the basis for disputing the violation, (2) the corrective steps that have been taken and the results achieved, (3) the corrective steps that will be taken to avoid further violations, and (4) the date when full compliance will be achieved. Your response may reference or include previously docketed correspondence, if the correspondence adequately addresses the required response. If an adequate reply is not received within the time specified in this Notice, an order or a Demand for Information may be issued as to why the license should not be modified, suspended, or revoked, or why such other action as may be proper should not be taken. Where good cause is shown, consideration will be given to extending the response time.

Because your response will be placed in the NRC Public Document Room (PDR), to the extent possible, it should not include any personal privacy, proprietary, or safeguards information so that it can be placed in the PDR without redaction. If personal privacy or proprietary information is necessary to provide an acceptable response, then please provide a bracketed copy of your response that identifies the information that should be protected and a redacted copy of your response that deletes such information. If you request withholding of such material, you must specifically identify the portion that you seek to have withheld and provide in detail the bases for your claim of withholding (e.g., explain why the disclosure of information will create an unwarranted invasion of personal privacy or provide the information required by 10 CFR 2.790(b) to support a request for withholding confidential commercial or financial information). If safeguards information is necessary to provide an acceptable response, please provide the level of protection described in 10 CFR 73.21.

Dated at Lisle, Illinois,  
this 15th day of July 1997

