

CATEGORY 1

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ACCESSION NBR: 9612020052 DOC. DATE: 96/11/25 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315
 AUTH. NAME AUTHOR AFFILIATION
 PISARSKY, F. American Electric Power Co., Inc.
 BLIND, A.A. American Electric Power Co., Inc.
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 96-006-00: on 961024, interim LER-one hours action statement requirements not met for inoperable Unit 1 power operated relief block valve. Caused by incorrect assumption. Preventive actions is expected to be submitted. W/961125 ltr.

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American Electric Power
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



November 25, 1996

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled Licensee Event Report System, the following report is being submitted:

96-006-00

Sincerely,

A handwritten signature in cursive script that reads 'A. A. Blind'.

A. A. Blind
Site Vice President

/mbd

Attachment

c: A. B. Beach, Region III
E. E. Fitzpatrick
P. A. Barrett
S. J. Brewer
J. R. Padgett
D. Hahn
Records Center, INPO
NRC Resident Inspector

9612020052 961125
PDR ADDCK 05000315
S PDR

020018

IE 22
1/1

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)
Donald C. Cook Nuclear Plant - Unit 1DOCKET NUMBER (2)
50-315

Page 1 of 1

TITLE (4)

Interim LER - One Hours Action Statement Requirements Not Met for Inoperable Unit 1 Power Operated Relief Block Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
10	24	96	96	-- 006 --	00	11	25	96	FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)			
POWER LEVEL (10)	89	20.2201(b)	20.2203(a)(3)(i)	50.73(a)(2)(iii)	73.71(b)
		20.2203(a)(1)	20.2203(a)(3)(ii)	50.73(a)(2)(iv)	73.71
		20.2203(a)(2)(i)	20.2203(a)(4)	50.73(a)(2)(v)	OTHER
		20.2203(a)(2)(ii)	50.36(c)(1)	50.73(a)(2)(vii)	(Specify in Abstract below and in Text, NRC Form 366A)
		20.2203(a)(2)(iii)	50.36(c)(2)	50.73(a)(2)(viii)(A)	
		20.2203(a)(2)(iv)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(B)	
	20.2203(a)(2)(v)	50.73(a)(2)(ii)	50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

TELEPHONE NUMBER (Include Area Code)

Frank Pisarsky, Production Engineering - Mechanical Components Supervisor

616/465-5901,x2607

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

X YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			12	20	96

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On October 24, 1996, at 1215 hours with Unit 1 in Mode 1 at 89 percent Rated Thermal Power, 1-NMO-152, the block valve for Power Operated Relief Valve (PORV) 1-NRV-152, was declared inoperable. It was subsequently determined that the valve had been inoperable since October 14, 1996, when the calculation run for 1-NMO-152 using a revised valve factor showed that the thrust required to close the valve under a design basis differential pressure was above the current thrust setting in the closed direction. As the valve was not declared inoperable until October 24, 1996, the 1 hour Action Statement requirements were not met. This interim LER is being submitted in accordance with 10CFR50.73(a)(2)(i)(B), to report a condition prohibited by the plant's Technical Specifications. The valve factor (0.4) used in the original calculation for 1-NMO-152, derived from EPRI test loop data of a similar valve, had been previously evaluated and accepted by the NRC. As part of the final evaluation the EPRI Performance Prediction computer model was to be run to determine a bounding valve factor. This run indicated a more conservative value of 0.51 should be used. However, this valve factor of 0.51 was not applied at the time the information was received, due to an incorrect assumption that the 0.4 valve factor was still applicable until an upcoming design change was implemented. When the use of the 0.4 was questioned during the Motor Operated Valve Close-out Inspection, the incorrect assumption was noted. The 0.51 valve factor was subsequently applied to the thrust calculation and the valve was declared inoperable.

The investigation into the root cause of this event continues, as does the evaluation to determine its safety significance. A final report, with a finalized root cause, safety significance and preventive actions, is expected to be submitted by December 20, 1996.