

PRIORITY 1

(ACCELERATED RIDS PROCESSING)

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSION NBR: 9508180077 DOC. DATE: 95/08/14 NOTARIZED: NO DOCKET #
 FACIL: 50-315 Donald C. Cook Nuclear Power Plant, Unit 1, Indiana M 05000315 P
 AUTH. NAME AUTHOR AFFILIATION
 NICHOLS, W. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele R
 BLIND, A.A. Indiana Michigan Power Co. (formerly Indiana & Michigan Ele
 RECIP. NAME RECIPIENT AFFILIATION

SUBJECT: LER 95-003-00: on 950714, RT occurred due to turbine trip on
 loss of condenser vacuum. Caused by broken weld on 1 inch
 drain trap bypass for steam dump valve 1-URV-110 into "A"
 condenser. Completed weld repair. W/950814 ltr. R

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 TITLE: 50.73/50.9 Licensee Event Report (LER), Incident Rpt, etc. I

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EXTERNAL:	L ST LOBBY WARD		1	1	LITCO BRYCE, J H	2	2	
	NOAC MURPHY, G.A		1	1	NOAC POORE, W.	1	1	
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Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616 465 5901



August 14, 1995

United States Nuclear Regulatory Commission
Document Control Desk
Rockville, Maryland 20852

Operating Licenses DPR-58
Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by
10 CFR 50.73 entitled Licensee Event Report System, the
following report is being submitted:

95-003-00

Sincerely,

A handwritten signature in black ink that reads 'A. A. Blind'. The signature is written in a cursive style.

A. A. Blind
Plant Manager

/clc

Attachment

c: H. J. Miller, Region III
E. E. Fitzpatrick
P. A. Barrett
R. F. Kroeger
M. A. Bailey - Ft. Wayne
NRC Resident Inspector
J. R. Padgett
G. Charnoff, Esq.
D. Hahn
Records Center, INPO
S. J. Brewer

9508180077 950814
PDR ADOCK 05000315
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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook Nuclear Plant - Unit 1	DOCKET NUMBER (2) 05000 315	PAGE (3) 1 OF 3
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TITLE (4)
Reactor Trip Due To Turbine Trip On Loss Of Condenser Vacuum

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
07	14	95	95	003	00	08	14	95	FACILITY NAME	DOCKET NUMBER
										05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)									
POWER LEVEL (10) 100%	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	<input type="checkbox"/> OTHER						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 368A)						
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)

NAME William Nichols, Operations Superintendent	TELEPHONE NUMBER (include Area Code) 616-465-5901 x2536
---	---

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At 0426 hours on July 14, 1995, while at 100 percent Rated Thermal power, Unit 1 received a reactor trip signal as a result of a turbine trip due to low condenser vacuum. The cause of the loss of condenser vacuum was a broken weld on the 1 inch drain trap bypass for steam dump valve 1-URV-110 into the "A" condenser. The break was at the socket to pipe weld joint and completely separated the pipe from the fitting, allowing air to be admitted into the "A" condenser and vacuum to decrease beyond the trip setpoint.

All safety systems operated normally in response to the trip signal. With the exception of the failure of bleed steam to feedwater heaters 5B and 6B to automatically isolate due to a defective relay, all post reactor trip responses were normal. The unit was stabilized in Mode 3, Hot Standby, and repairs were performed. This event was determined to have no actual or potential adverse effect on the health and safety of the public.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-830), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		95	0 0 3	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 368A's) (17)

Conditions Prior to Occurrence:

Unit 1 in Mode 1 (Power Operations) at 100 percent Rated Thermal Power near the end of the fuel cycle.

Description of Event

At 0426 hours on 7-14-95 while at 100 percent power, Unit 1 received a reactor trip as a result of a turbine trip due to low condenser vacuum. The cause of the loss of condenser vacuum was a broken 1 inch line on the drain trap bypass for steam dump valve 1-URV-110 into the "A" condenser. The break was at the socket to pipe weld joint and completely separated the pipe from the fitting allowing air to be admitted into the "A" condenser.

Increased Steam Jet Air Ejector (SJAE) flows were seen prior to the trip. The SJAEs were in a normal line-up, with the SJAE air offtake cross-tie valves closed and the "A" condenser and the feedpump condensers connected to the same SJAEs. Vacuum was lost first in the "A" condenser, then in the West Feedpump condenser and finally in the East Feedpump condenser.

Eight minutes elapsed from the time of the condenser "A" low vacuum alarm was received at 24.3" Hg vacuum and the turbine trip at 21.8" Hg vacuum. During this time the operators attempted to pinpoint the source of the vacuum loss, by verifying alignment of the steam seals, SJAEs, Circulating Water Pumps, condenser vacuum breakers and condenser water boxes. All line-ups were normal and the source of the inleakage could not be located.

After the reactor trip condenser vacuum slowly returned to its normal value due to steam flow in 1-URV-110, which stopped the flow of air into the "A" condenser.

It was noted, after the trip, that bleed steam to feedwater heaters 5B and 6B did not automatically isolate and were manually closed. Turbine trip relay 1-5-TVT4, which serves to close bleed steam valves 1-HMO-502 and 1-HMO-602, was found to be defective and replaced. All other post reactor trip responses were normal, and the unit was stabilized in Mode 3, Hot Standby.

Cause of the Event:

Fatigue related mechanisms, due to vibration, initiated a crack at the toe of the socket weld connecting the 1" pipe to the socket fitting attached to the large-bore pipe. Further propagation of the crack caused the 1" pipe to separate from the socket. No evidence of corrosion or flow-assisted corrosion was evident at the location of the failure.

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Donald C. Cook - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		9 5	- 0 0 3	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Analysis of Event:

This event is being reported per 10 CFR 50.73(a)(2)(iv) as an event that resulted in automatic actuation of Engineered Safety Features (ESF), including the Reactor Protection System (RPS).

A reactor trip occurred when the main turbine tripped above 10 percent Rated Thermal Power on a Main Turbine Low Condenser Vacuum trip. All control rods fully inserted, both Motor Driven Auxiliary Feedwater Pumps started, and a feedwater isolation occurred; all as designed.

Normal offsite power was available and the emergency diesel generators were in standby. The East Containment Spray Pump was being returned to an operable status prior to the trip, and no other safety equipment was out of service. This event did not have any actual or potential adverse impact on the health and safety of the public.

Corrective Action:

The weld repair for the bypass line around 1-URV-110 was completed and turbine trip relay 1-5-TVT4 was replaced on July 17, 1995. A liquid penetrant examination was performed on the similar lines for the other eight steam dump valves in Unit One. During this inspection, a surface crack was discovered on a weld for the bypass line of 1-URV-112. This weld was repaired in a similar manner as the weld on URV-110. No other defects were noted by the penetrant exams.

The Unit Two steam dump bypass lines were also inspected. A preliminary visual exam to accessible areas did not detect any areas of concern, however, a more detailed surface exam will be conducted during the next Unit Two Refueling outage.

Failed Component Identification:

N/A

Previous Similar Events:

None