## ACCELERATED DISTRIBUTION DEMONSTRATION SYSTEM

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

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ACCESSION FACIL:50 AUTH.NA WEBER,G BLIND,A	N NBR:9210280001 D-315 Donald C. Coc AME AUTHOR A .A. Indiana M .A. Indiana M	DOC.I bk Nuc AFFILIA Aichiga Aichiga	DATE: 9 Lear Po ATION an Powe an Powe	92/10/23 NOTARIZED ower Plant, Unit 1, er Co. (formerly In er Co. (formerly In	: NO Indiana M diana & Mi diana & Mi	DOCKET # 05000315 chigan Ele chigan Ele
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	submitted to incl	ease	ed by s tolerai	nce limits.W/921023	ltr.	e D
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## NOTE TO ALL "RIDS" RECIPIENTS:

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Indiana Michigan Power Company Cook Nuclear Plant One Cook Place Bridgman, MI 49106 616 465 5901

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October 23, 1992 \*\*

United States Nuclear Regulatory Commission Document Control Desk Rockville, Maryland 20852

> Operating Licenses DPR-58 Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10 CFR 50.73 entitled <u>Licensee Event Report System</u>, the following report is being submitted:

92-006-01

Sincerely,

Á. A. Blind

Plant Manager

/sb

Attachment

C:	D. H. Williams, Jr.
	A. B. Davis, Region III
	E. E. Fitzpatrick
	P. A. Barrett
	R. F. Kroeger
	B. Walters - Ft. Wayne
•	NRC Resident Inspector
	J. F. Stang - NRC
	J. G. Keppler
	M. R. Padgett
	G. Charnoff, Esq.
	D. Hahn
	INPO
	S. J. Brewer/B. P. Lauzau
	B. A. Svensson

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NRC FOR	NRC FORM 366 U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104																	
	EXPIRES: 4/30/92 EXPIRES: 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH TH INFORMATION COLLECTION REQUEST: 50.0 HRS, FORWAR COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORD AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLE/ REGULATORY COMMISSION, WASHINGTON, DC 20555, AND THE PAPERWORK REDUCTION PROJECT (315:04)104), OFFIN OF MANAGEMENT AND BURDET, WASHINGTON, DC 20503.									THIS WARD ORDS LEAR ND TO FFICE								
FACILITY	ACILITY NAME (1) DOCKET NUMBER (2) PAGE (3)										(3)							
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value was out of tolerance by 6.23 percent (67 psig).																		
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1	cha	ange will	minimize	the num	ber c	of MSS	SV fai	611	ures	and	is d	cor	isistent	: wit	h.			
1	revised testing standards developed by ANSI OM-1 Committee.																	
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NRC FORM 366A (6-89)		U.S; NU	ICLEAR REGULATOR	Y COMMISSION		APPROVED	OMB NO. 3150-01	04	
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This supple	mental repor	t is being	submitted	to provid	de add:	itional c	ause and		
COLLECTIVE	action into:	Indcion.						I	
<u>Conditions</u>	Prior to Occ	urrence:	I						
Unit One -	49 Percent R	eactor Ther	mal Power						
Description	of Event:								
On June 18	and 19, 1992	, ten of th	e twenty M	ain Stear	m Safe	ty Valves	(MSSVs)		
(EIIS/SB-RV	7) lift setti	ngs were fo	ound outsid	le of the	+/- 01	ne percen	t tolerar	ice	
out-of-tole	iblisnea in u France MSSVs	were all fo	und to lif	t at valu	ues wh:	ich excee	ded the		
upper toler	ance (+1 per	cent) limit	Six of	the MSSV	s had	lift valu	es that		
were out-of	-tolerance b	y 16 psig c	or less. T	hree of t	the MS	SVs had I ther MSSV	ift value /g lift.	28	
of greater than 3 percent (between 34 and 39 psig). The other MSSV's lift value was out-of-tolerance by 6.23 percent (67 psig).									
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NRC FORM 36	6A U.S	NUCLEAR REGULATORY COM	MISSION	APPROVED OMB NO. 315	0-0104
(6-89)				EXPIRES: 4/30/92	
	LICENSEE EVENT REPORT	(LER)		INFORMATION COLLECTION REQUEST:	50.0 HRS, FORWARI
•	TEXT CONTINUATION			AND REPORTS MANAGEMENT BRANCH REGULATORY COMMISSION, WASHINGT	(P-530), U.S. NUCLEAI ON, DC 20555, AND T
	•			OF MANAGEMENT AND BUDGET, WASHI	NGTON, DC 20503.
FACILITY NA	AE (1)	DOCKET NUMBER (2)		LER NUMBER (6)	PAGE (3)
				YEAR SEQUENTIAL REVISION	
D. C.	COOK NUCLEAR PLANT - UNIT 1	0 5 0 0 0 3	1 5	92 -0 06 -01	030F0
TEXT (If more a	pace is required, use additional NRC Form 366A's) (17)	u			1
	Analysis of Event:		,		*
	The safety valve lift setpoints compliance with the Technical Sr	reported here we ecification (T/)	ere f( 5) 3.1	ound to be out of 7.1.1 requirements and	L
	therefore reportable per 10CFR50	.73(a)(2)(i)(B)	Th	e as-found condition c	f
1	the MSSVs did not have any impac	t on the health	or s	afety of the public.	
	Out of the ten Main Steam Safety	Valves, with h	lgh 1	ift setpoints, six ope	ned
-	between 1 and 3 percent above th	e Technical Spec	cific	ation Limit. The	2
	remaining four had lift setpoint percent. While all ten of these	s that were out valves violated	or-t d the	operability condition	lof
	Technical Specification 3.7.1.1,	an analysis has	bee	n performed by	
	Westinghouse to allow increasing	the setpoint to	olera	nce from +/- 1 percent	; xfa
	Technical Specification change.	WIII DE SUDMIC	leu c	o the ake in support o	
			.* <b>c</b>		
-	Based on the new Westinghouse ar small break LOCA events are the	alysis, the lost limiting transit	3 OI ents '	with respect to the	
	as-found MSSV lift setpoint valu	es.		·····	
	In the case of the loss of load	turbing trin ev	ont.	the greatest demand or	the
	MSSVs is created. Based on the	Westinghouse and	alysi	s, the steam relief	
	capacity of 8000 ft3/sec would h	e required to co	ompen	sate for the transient	*
	The MSSVs available (16 MSSVs wi Setpoint) would have provided su	fficient relief	capa	city (10,304 ft3/sec).	
Į	The full flow capacity of each w	alve is 238 1bm	/sec	at 1186.5 psig or	
	about 644 ft3/sec. Therefore, t	he 16 MSSVs, wh:	ich o	pened within 3 percent barge capacity of	. OÍ
	10,304 ft3/sec of steam. An una	cceptable press	ire b	uild-up in all the fou	ir . '
	steam generators would not have	occurred, since	two	or more valves in each	1
	steam generator would have opene	a within 3 perce	ent o	r their setpoints.	
	In the case of the small break I	OCA analysis, th	ne se	condary system flow as	.ds
	in the reduction of RCS pressure	. The primary :	inter	est for the MSSVs is	
	within the 3 percent range of th	e lowest Technic	cal S	pecification setpoint	
	(1065 psig) will provide suffici	ent flow to all	ow th	e pressure to remain	
ł	within the analyzed bounds. Aga would have operated within the 3	in, two or more percent range:	ther	es per steam generator efore, this condition	will
	be met.	Ference randol			
	The as-found MSSV setuciate would	d not have an a	ivera	e impact op the Reacto	)r
· ·	Coolant System (RCS) overpressu	e protection or	Depa	rture from Nucleate	-
1	Boiling (DNB) ratio. The RCS is	protected from	over	pressure conditions by	the
	rressurizer Safety valves and Po Steam Generator Power Operated H	wer operated Re. Selief Valves ca	n pe	used for RCS heat remo	oval.
		• • • •	-	handaren and Arrest !	
1	The LOCA long term cooling, hot	leg switchover,	LOCA Safet	DIOWOOWN, and Contain V Valves, and are	ment
	therefore not impacted.			· ····································	

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NRC FORM 366A (6-39)	U.S. NUCLEAR REGULATORY COMMISSI	APPROVED OMB NO, 3150-0104					
LICENSEE EVENT TEXT CONTINU	REPORT (LER) UATION	EXTINATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS, FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.					
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)					
		YEAR SEQUENTIAL REVISION NUMBER					
D. C. COOK NUCLEAR PLANT - UNI	T 1 0  5  0  0  0  3   1	5 9 2 - 0 0 6 - 0 1 0 4 OF 0 4					
TEXT (If more space is required, use additional NRC Form 306A's) (17)							
N, Carlos Car		r -					
<u>Corrective Action</u> :	,						
All Safety Valves found w ranges were reset to acce	ith lift setpoints outside ptable values and retested	e the acceptable setpoint a satisfactorily.					
The MSSVs that were out o and refurbished during th would explain the excessi Chief Engineer was brough Plant activities associat Representative reviewed t activities meet all curre for changes to the Plant	of tolerance by more than a e Refueling Outage. No de ve lift setpoints. The Dr t on-site to perform an Er ed with the MSSVs. The Dr he MSSV Test and Repair Pr ent Dresser practices. No MSSV Testing Program or re	B percent were disassembled eficiencies were noted that resser Valve Manufacturing ngineering Review of the resser Engineering rocedures. The Plant recommendations were made epair activities.					
A database has been estab repairs and test performa	lished to allow trending once.	of the individual MSSV					
The MSSV repair procedure critical valve components	s have been revised to do , whenever repairs are neg	cument the dimensions of cessary.					
Based on ANSI OM-1 Commit currently being taken to MSSV lift setpoint tolera	tee Safety Valve Test Request a change to Techning from one percent to the	virements, steps are ical Specification 3.7.1.1 wree percent.					
Failed Component Identifi	cation:	u I					
Main Steam Safety Valve Manufacturer: Dresser Co Model: 3707RA-RT22 EIIS Code: SB-RV	onsolidated Valves	т					
Previous Similar Events:							
50-315/90-13       5         50-315/89-02       5         50-315/87-11       5         50-315/86-20       5	0-316/92-03 0-316/90-06 0-316/88-04	• , • •					
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